

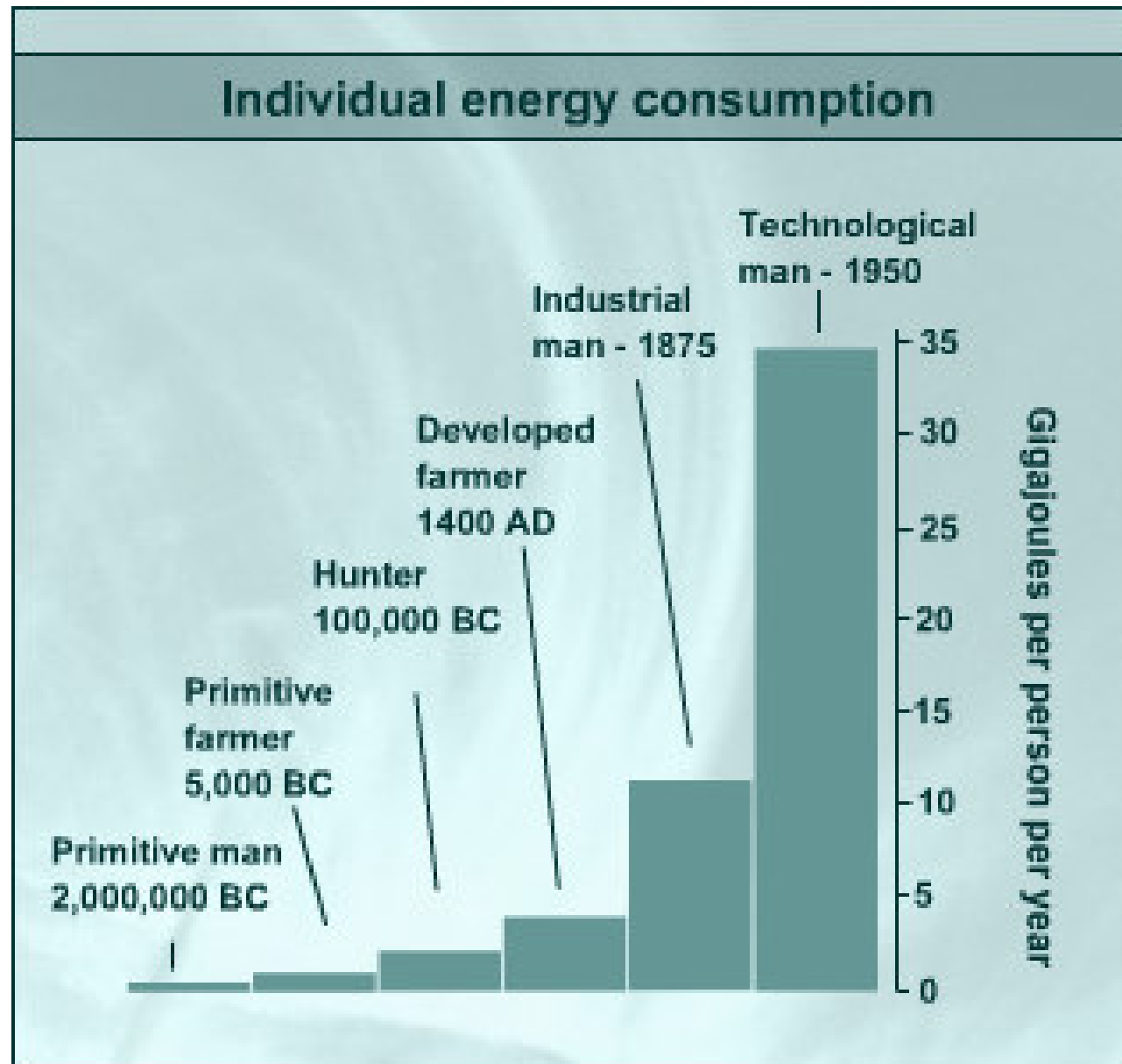
Soft X-ray spectrometer for plasma experiment Wendelstein 7-X

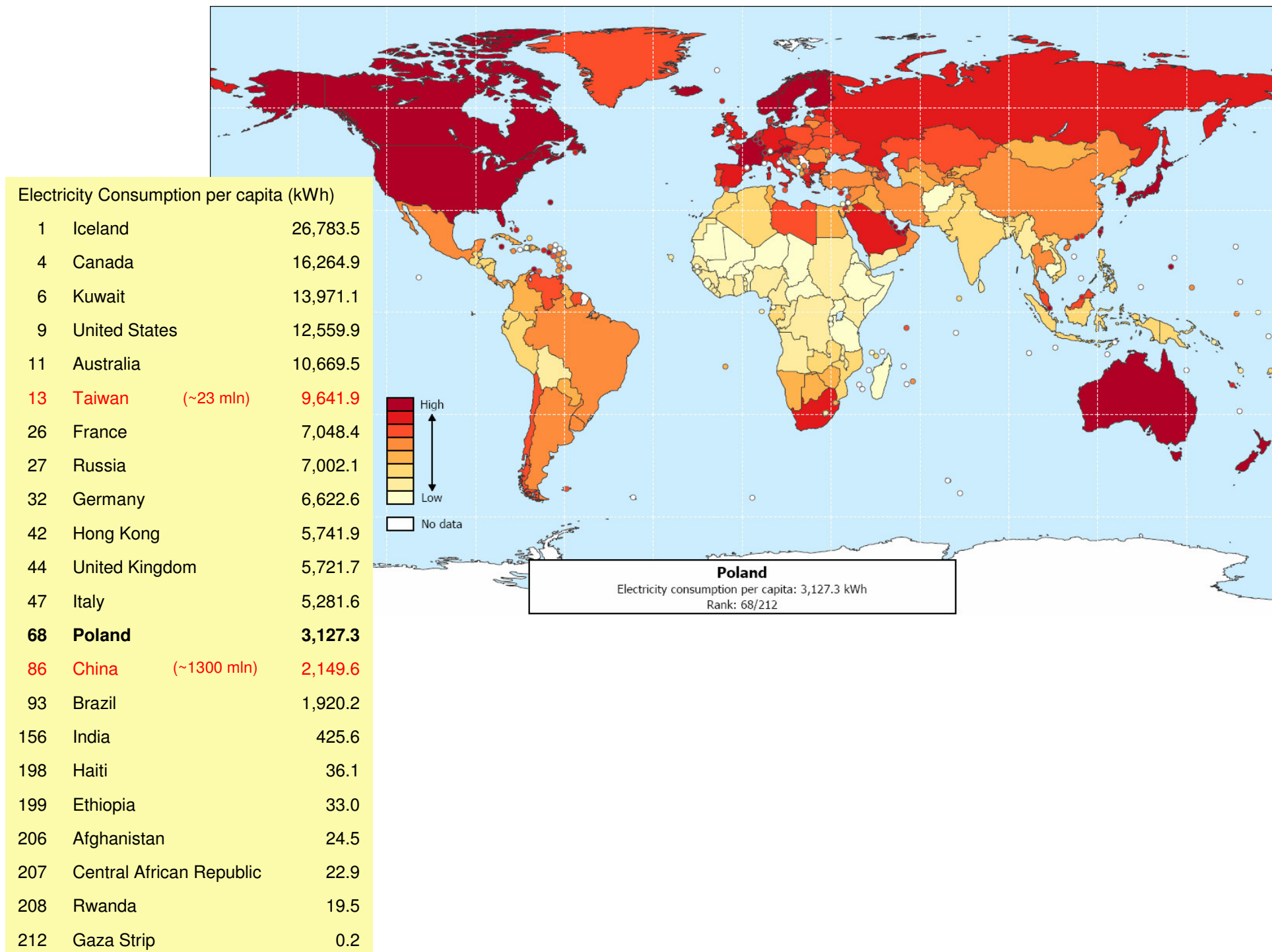
*Dr. Ireneusz Książek,
Institute of Physics, Opole University, Opole, Poland*

Outline:

- Thermonuclear fusion
- Magnetic confinement:
tokamak and/or/vs stellarator
- Wendelstein 7-X
- Soft X-ray radiation of high-temperature plasma
- C-/O- monitor for ASDEX-U
- C-/O- monitor for W7-X
- Detector for C-/O- monitor

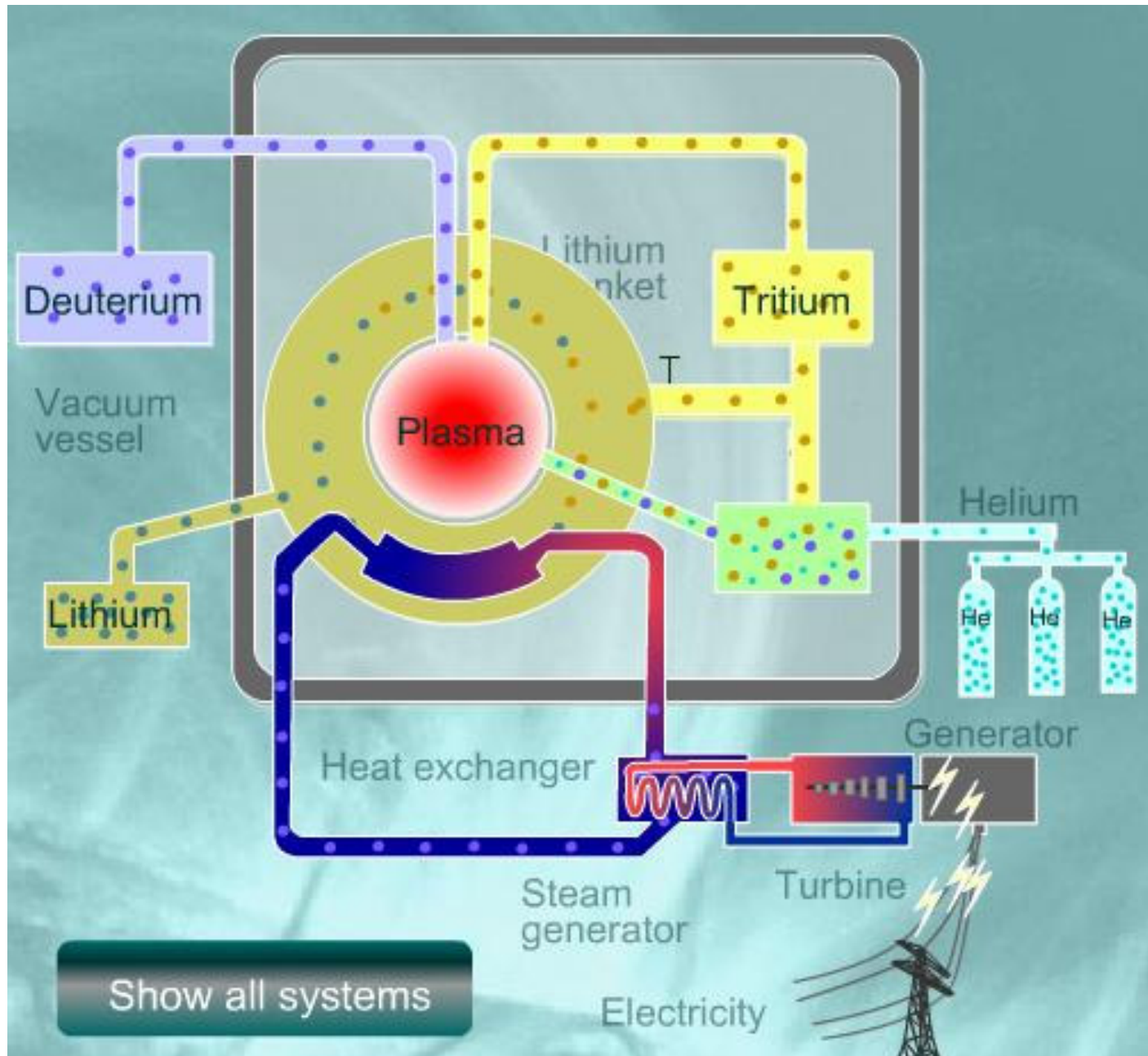
Energy demand



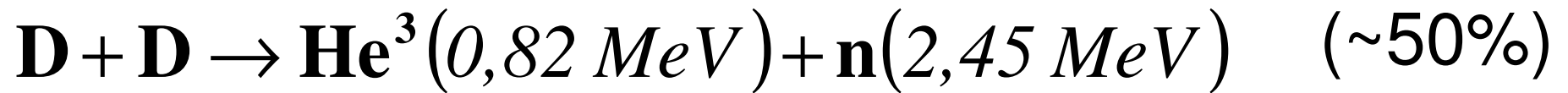
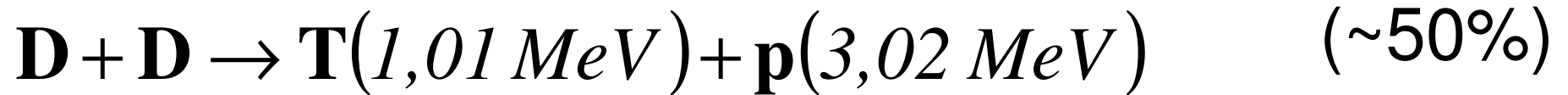


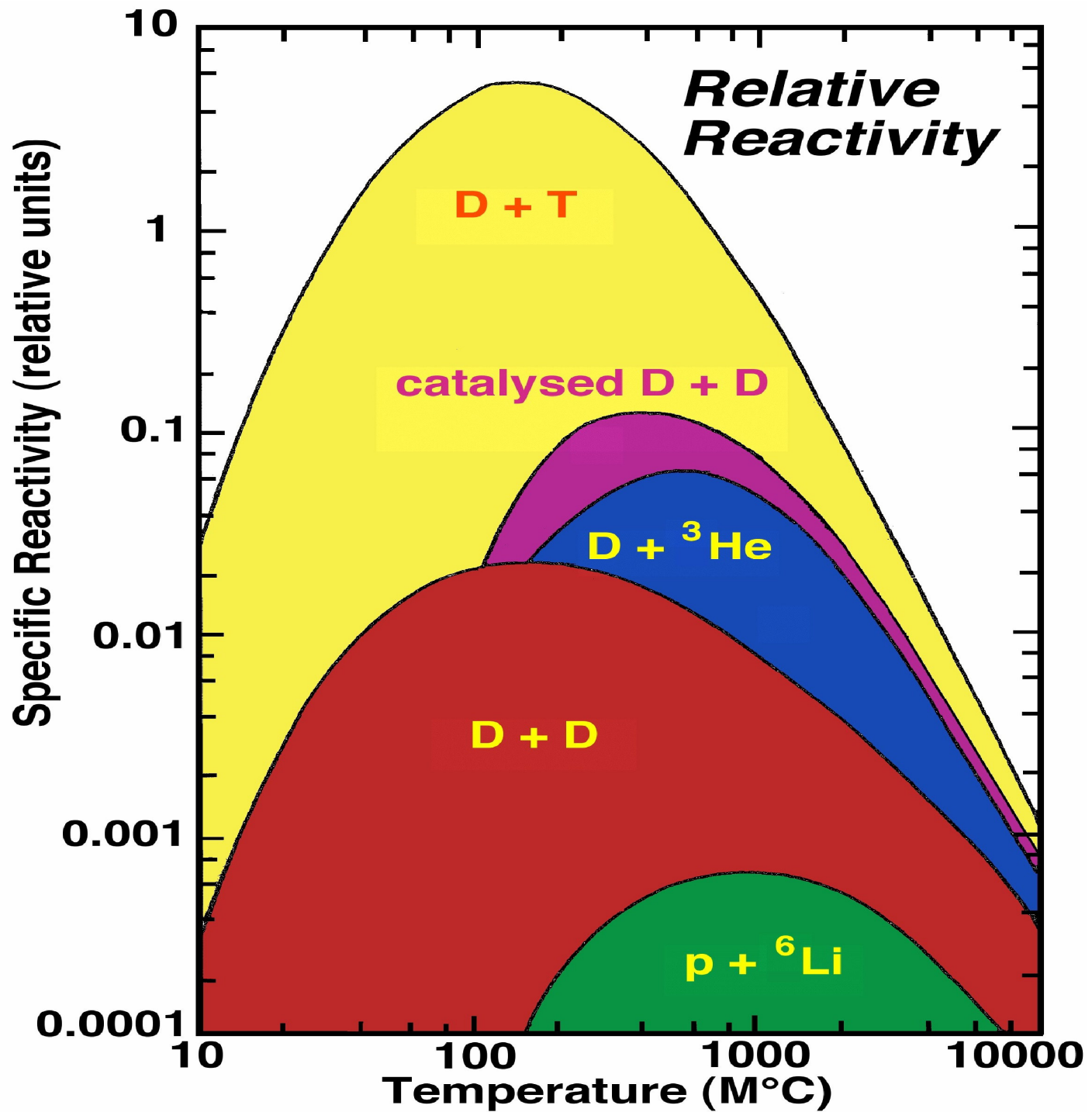
Source of energy – high temperature plasma

6



Fusion reactions:

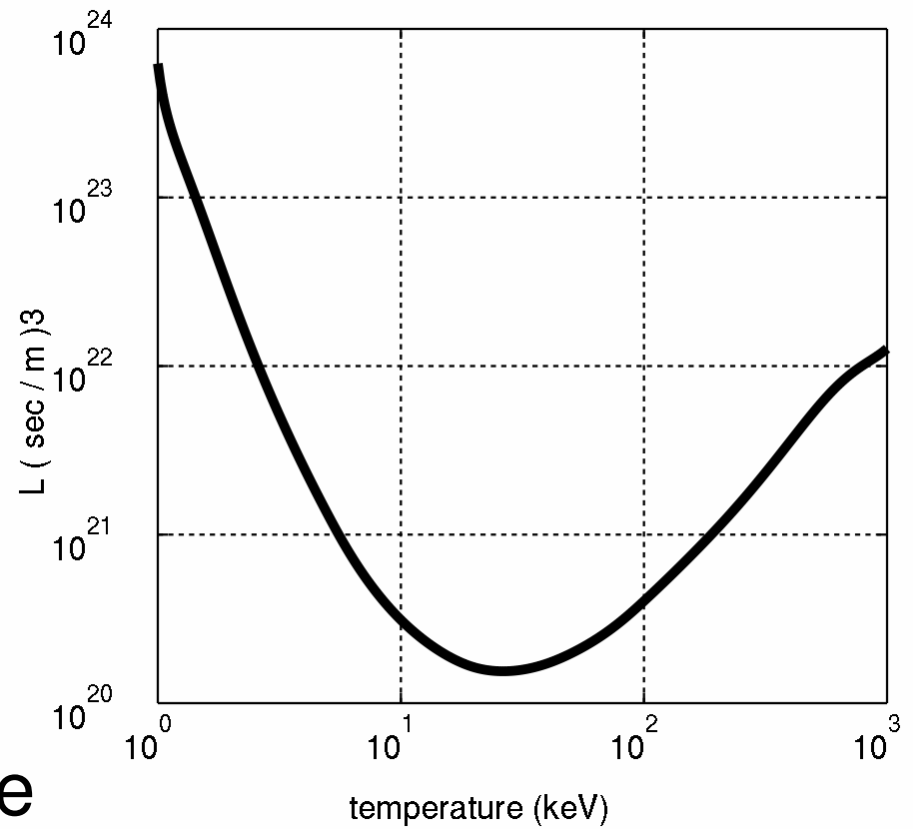




Magnetic confinement fusion

tokamak vs. stellarator

Lawson criterion (for D-T plasma)



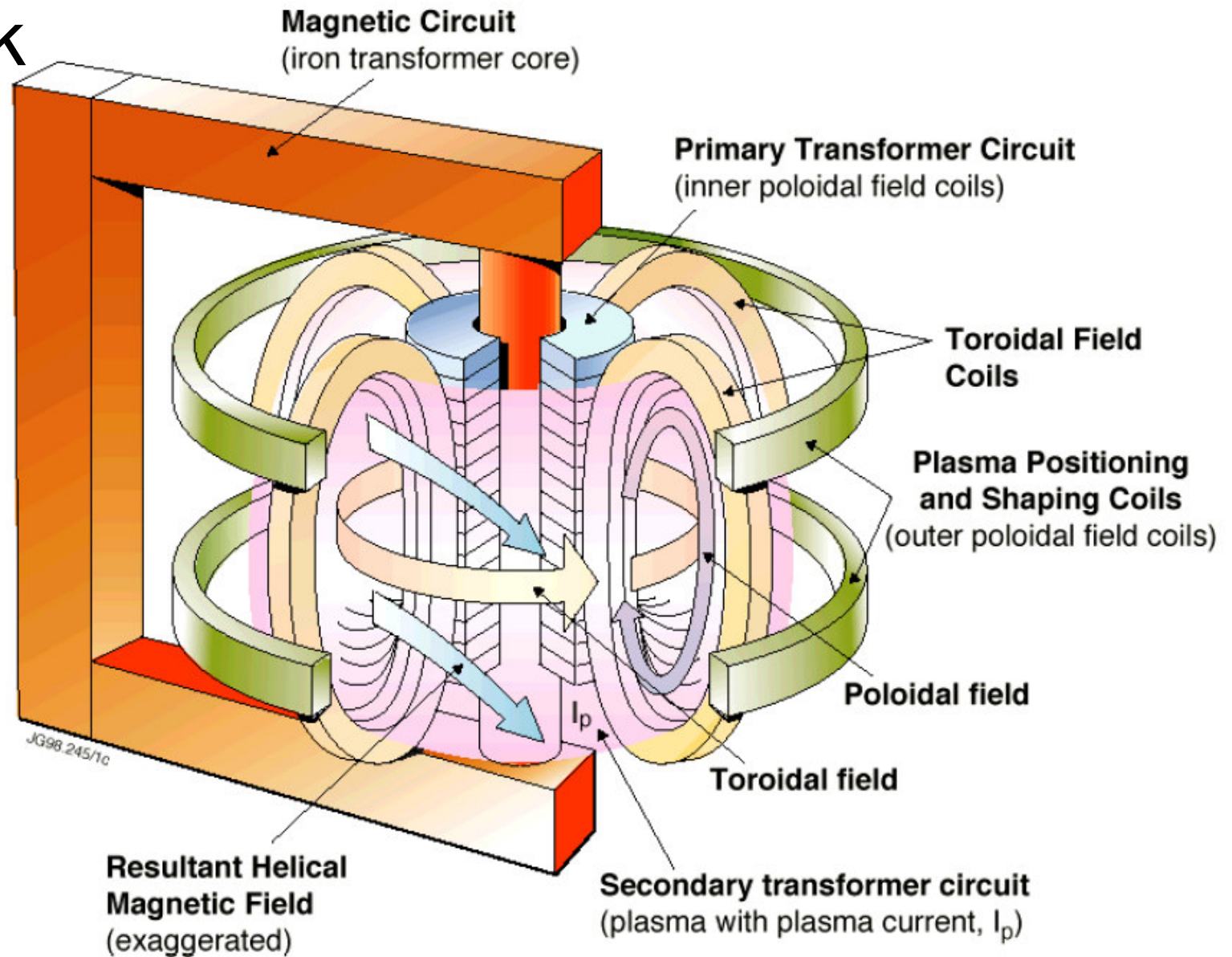
temperature

density

confinement time

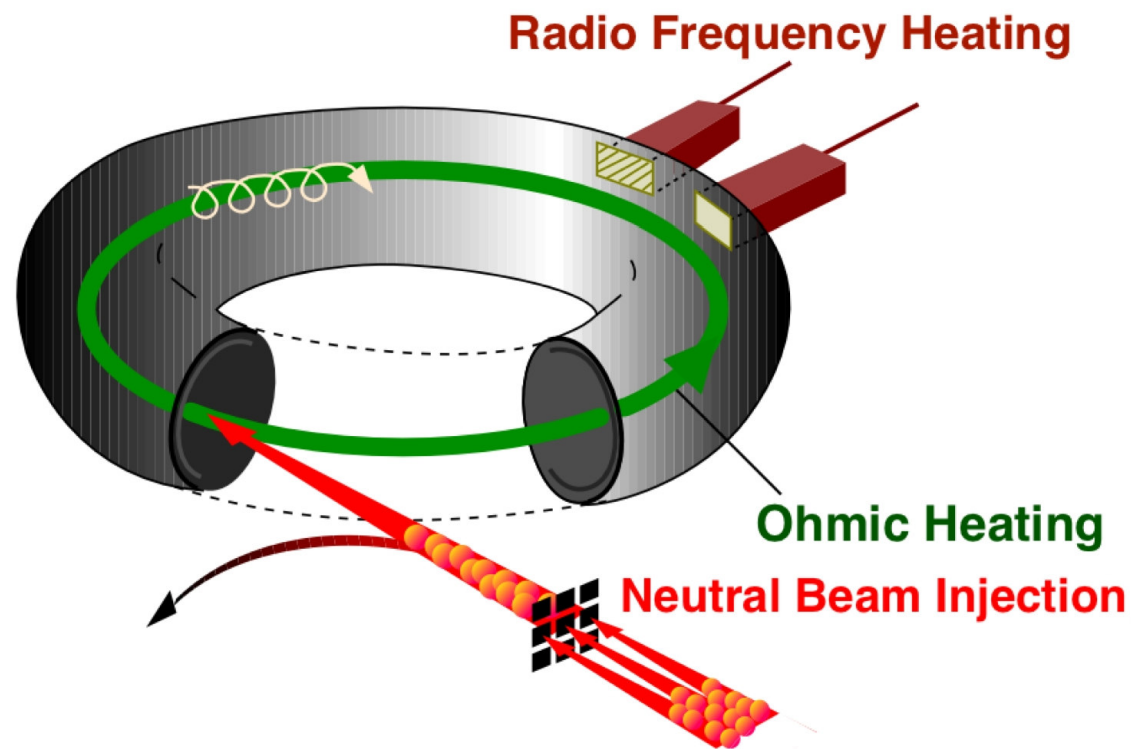
$$n \cdot T \cdot \tau > 10^{21} \text{ keV} \cdot \text{s/m}^3$$

Tokamak



$$n \approx 10^{20} m^{-3}, \quad \tau \approx 0.1 \div 100 s$$

Plasma heating



JET

(Joint European
Torus)

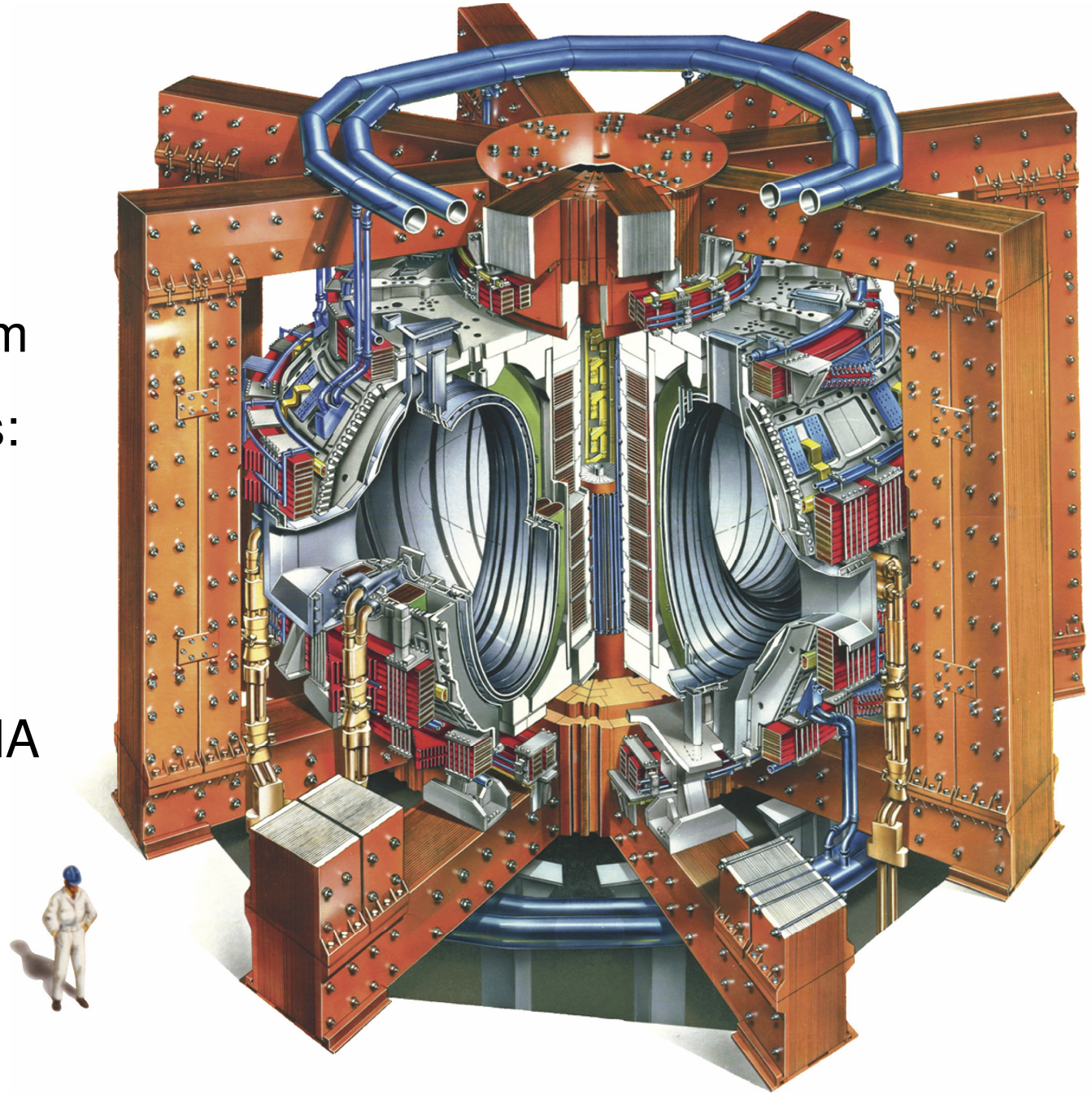
Major radius: 2,96 m

Plasma dimensions:
2,10 m × 1,25 m

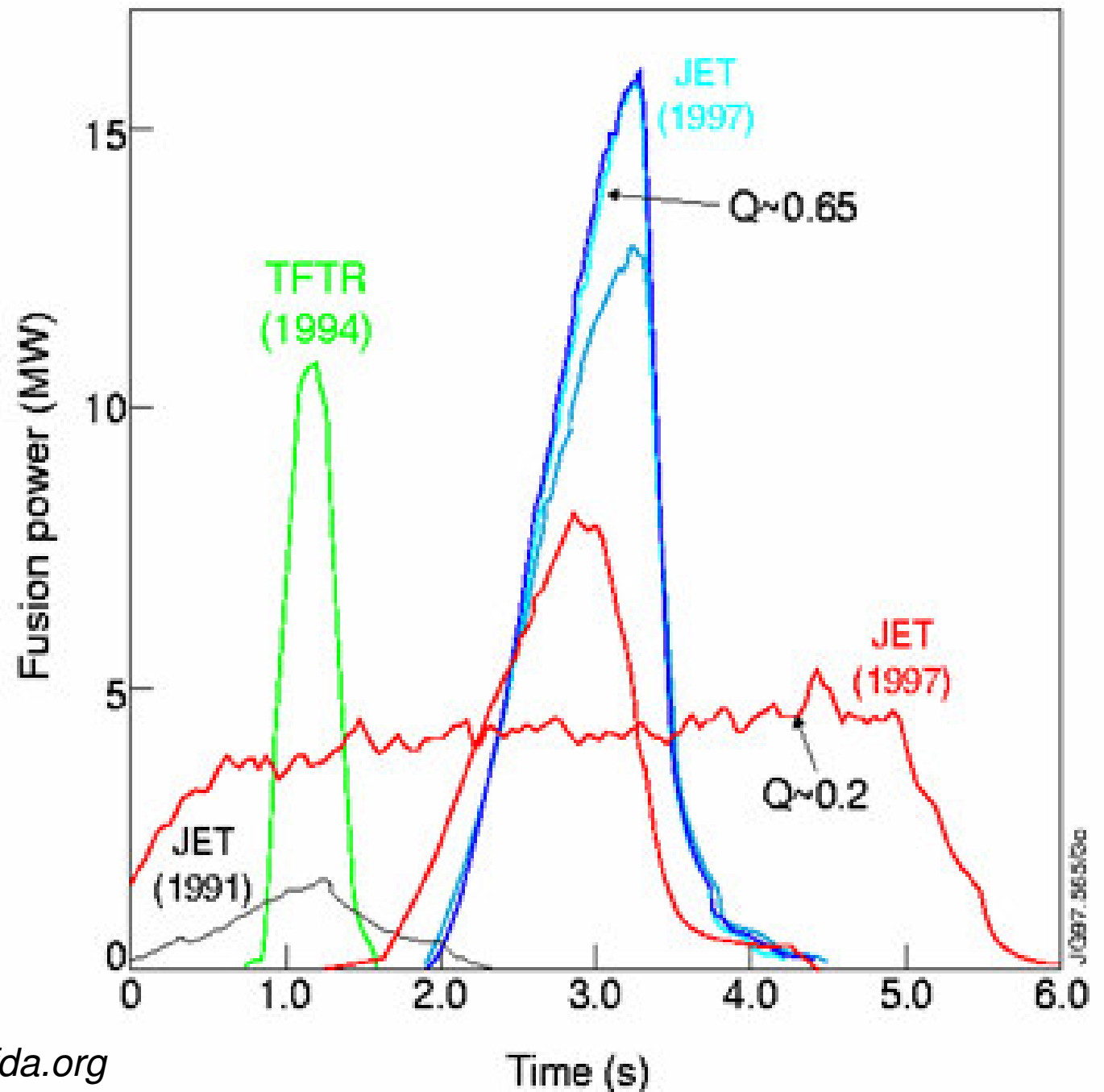
Plasma volume
(approx.) 100 m³

Plasma current 5 MA

Magnetic field: 4 T



JET best result



TOKAMAK problems

Only pulsed mode

Instabilities:

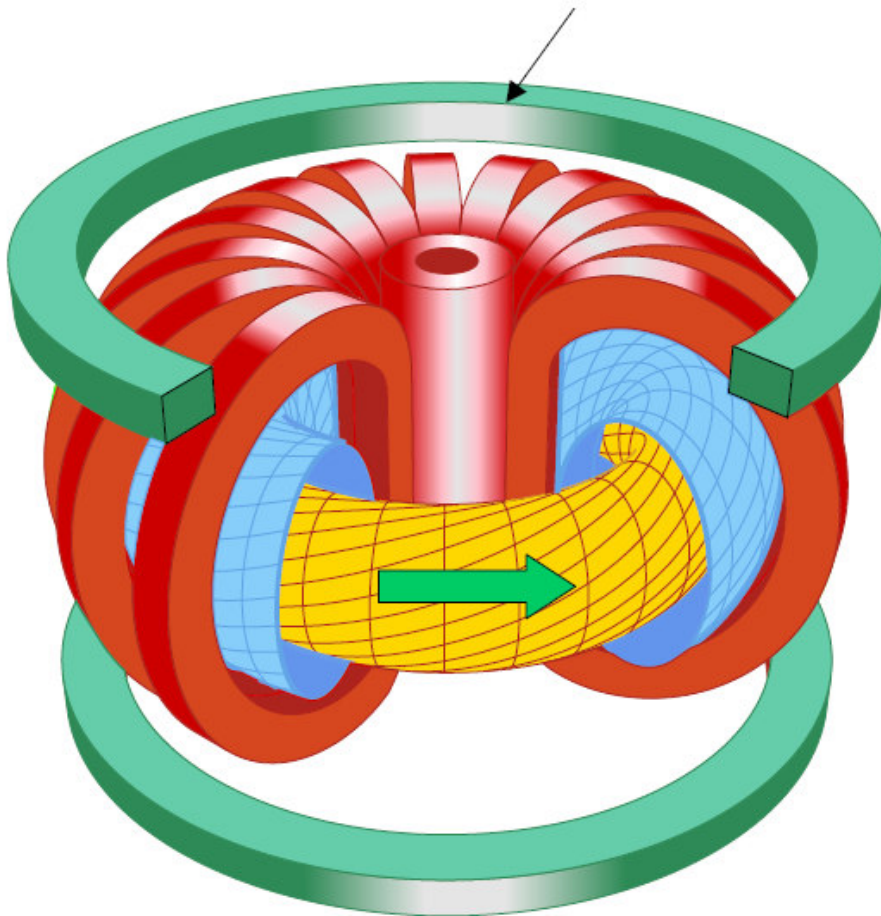
- ‘ELMs’ (Edge Localized Modes)
- ‘disruptions’

Problems to solve:

- transport theory
- material research

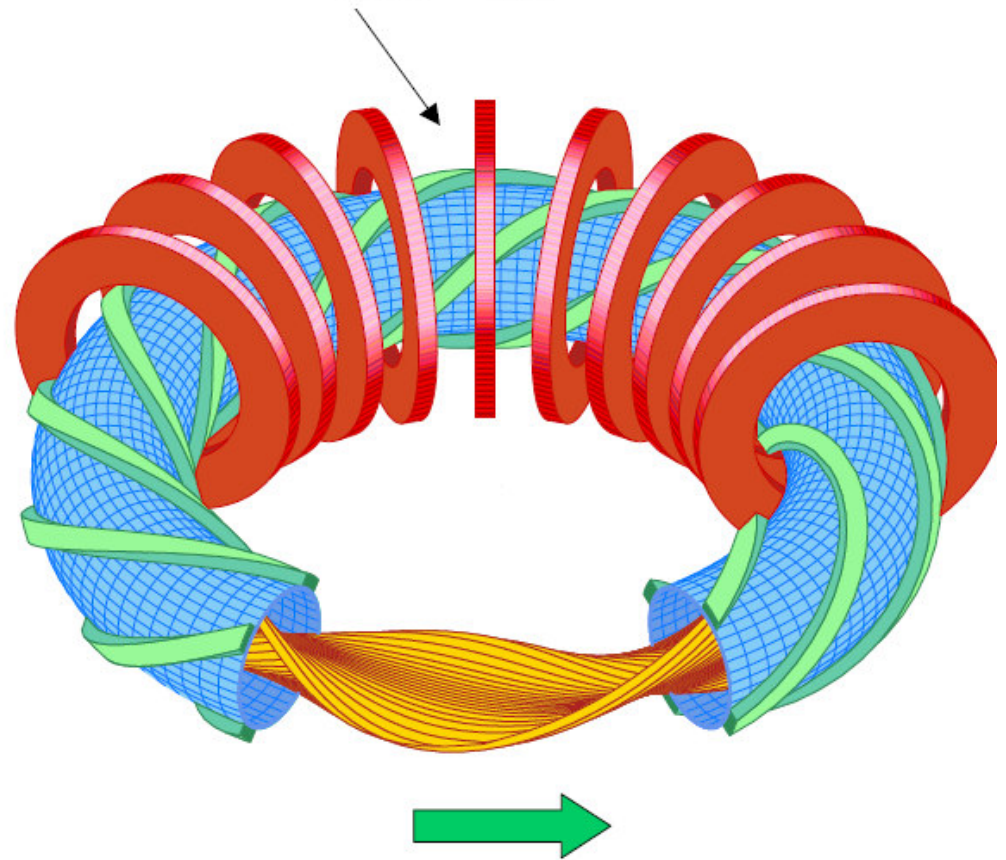
Tokamak or stellarator?

L. Artsimovich in Moskau about 1950



current flows in the plasma

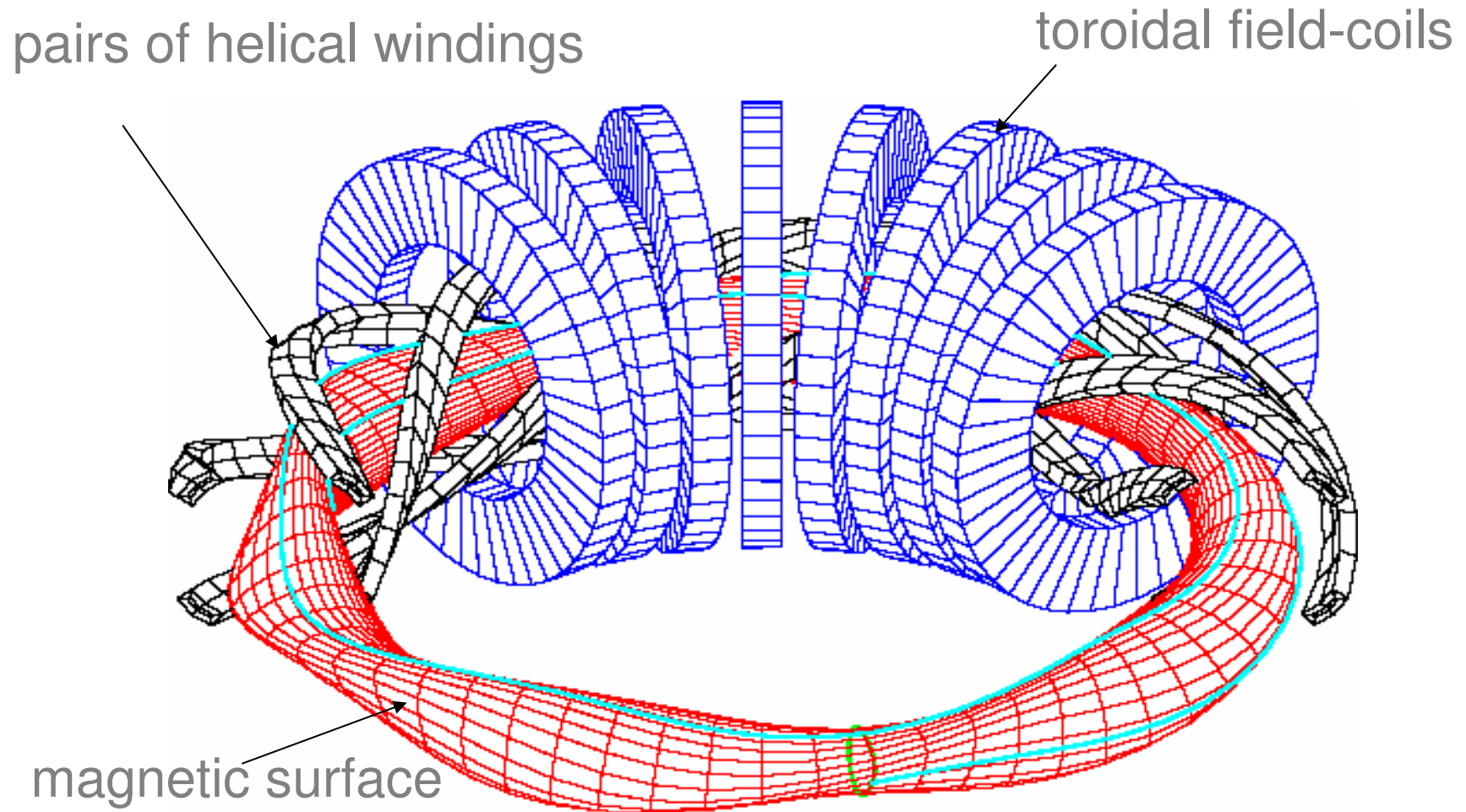
L. Spitzer in Princeton about 1950
stella= "star"



current in external (helical) coils

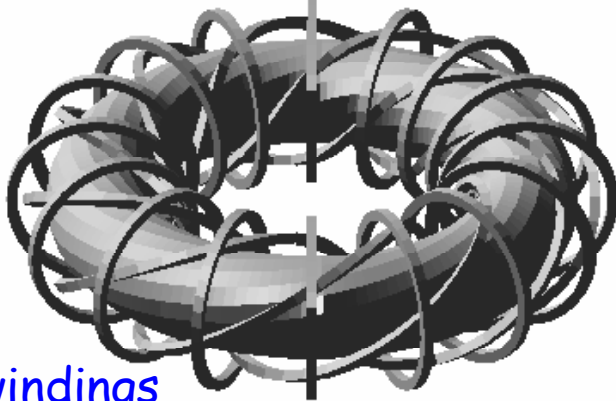
STELLARATOR, basic principle

17

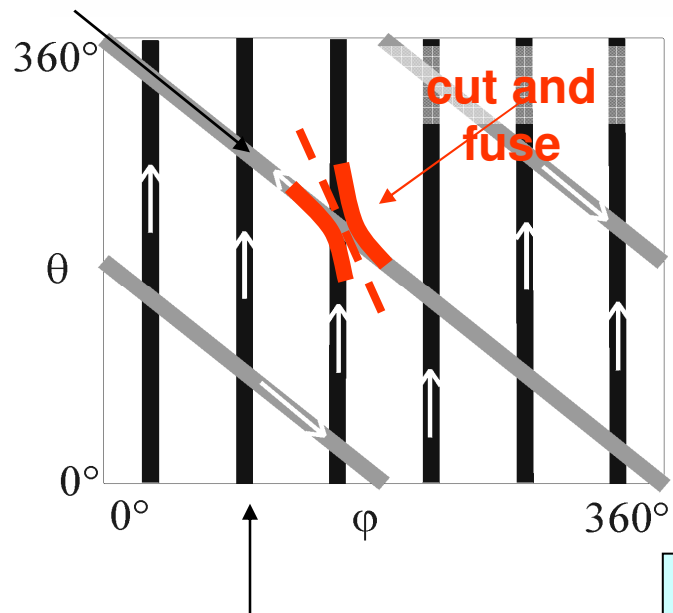


W7-X REALIZATION, modular coils

classical Stellarator

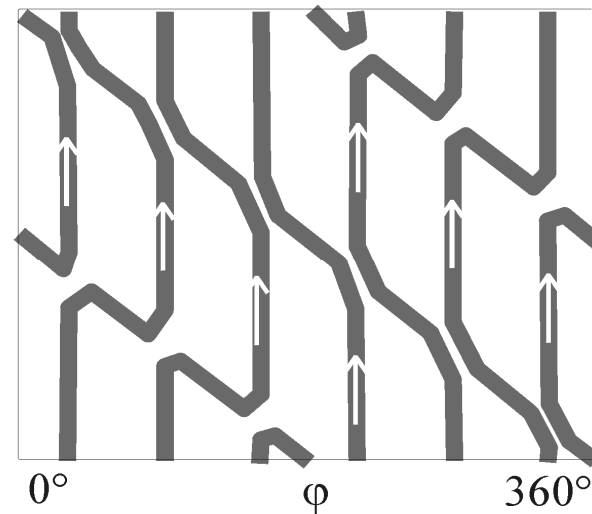
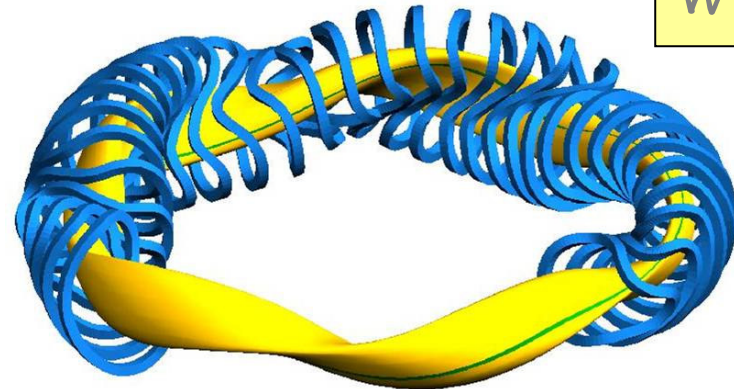


helical windings



toroidal field coil

W7-X



modular coils instead helical windings
avoidance of toroidal current: 3-d
geometry

Main purposes of Wendelstein 7-X:

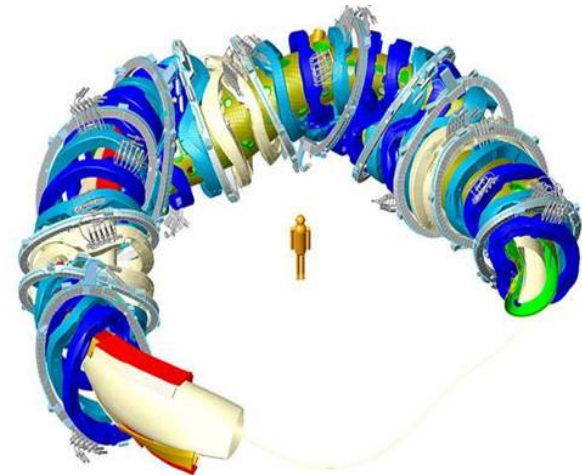
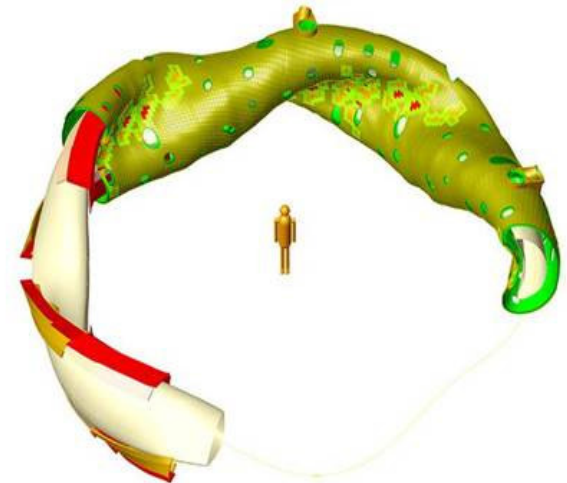
- producing plasmas allowing predictions on a stellarator power plant plasma
- continuous operation (up to 30 min.)
- constructing a modular superconducting coil system

W7-X, mechanical parameters

20

major radius	5.6 m
minor radius	0.53 m
plasma volume	30 m ³
induction on axis	3 T (2.5 T)
non-planar coils	50
planar coils	20

machine height:	4.5 m
machine diameter:	16 m
machine mass:	725 t



W7-X, heating, parameter range

$$T_e \leq 10 \text{ keV}$$

(electron temperature)

$$T_i \leq 6 \text{ keV}$$

(ion temperature)

$$n_e \leq 3 \times 10^{20} \text{ m}^{-3}$$

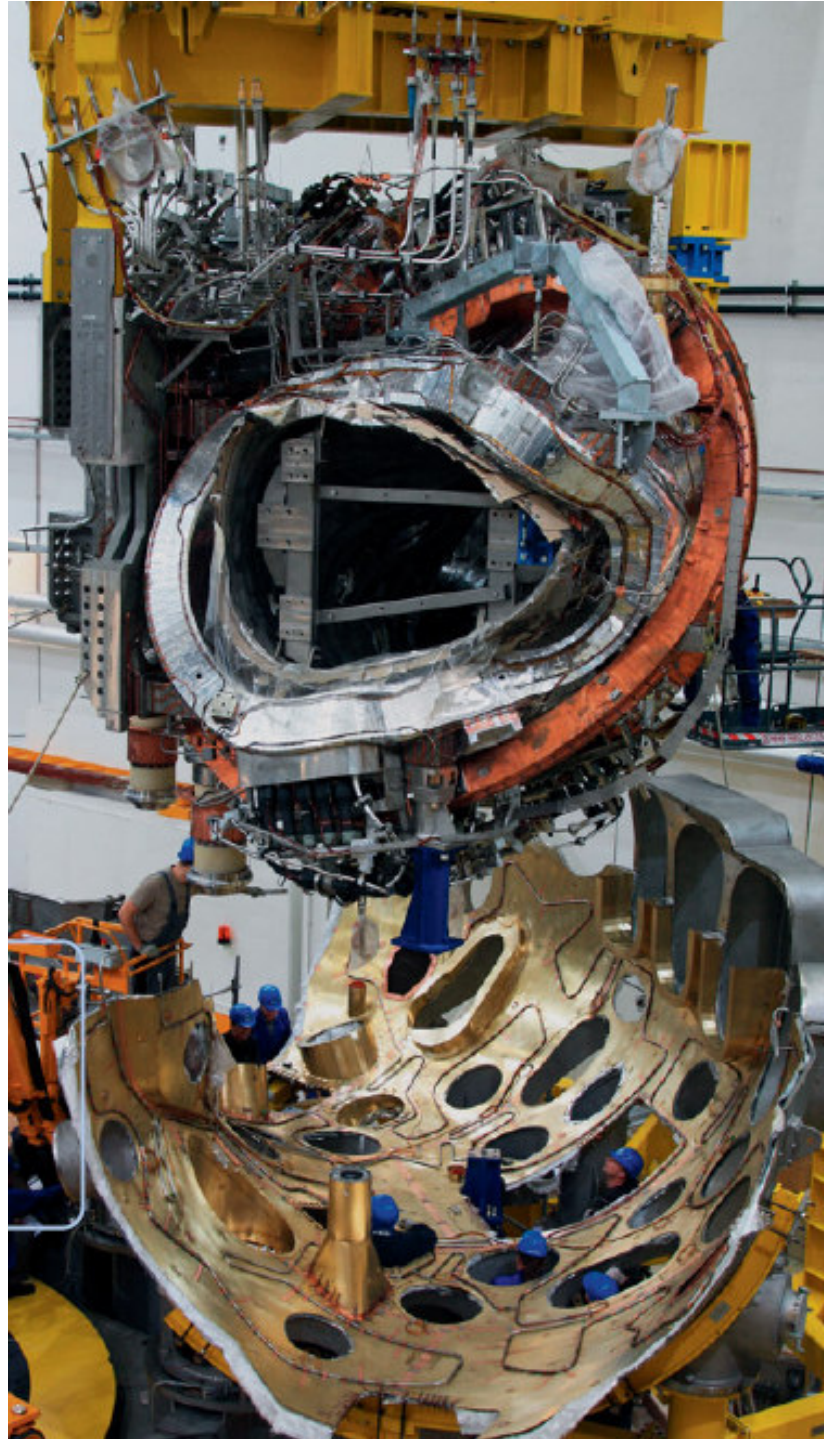
(electron density)

$$1 \text{ eV} = 11\,604 \text{ K}$$

- ECRH: 10 MW, 140 GHz, 1800 s
(Electron Cyclotron Resonance Heating)
- NBI: 10 MW 10 s
(Neutral Beam Injection)
- ICRH: 2x2 MW, 30-115 MHz, 10 s
(Ion Cyclotron Resonance Heating)

TEST ASSEMBLY, non-planar coil onto vacuum vessel²²





C-/O- Monitor

Measurements of most prominent lines (e.g. Lyman- α of hydrogen-like ions) of plasma impurities provide information about

- the erosion of the walls (C VI at 3.4 nm);
- the quality of the wall condition (O VIII at 1.9 nm);
- leakage in the vacuum systems (N VII at 2.5 nm);
- possible interaction of the plasma with materials containing boron (line B V at 4.9 nm);
- accumulation of impurities in plasma.

Monitor for the carbon and oxygen impurities in the ASDEX Upgrade tokamak

R. Neu, K. Asmussen, G. Fussmann, P. Geltenbort, G. Janeschitz, K. Schoenmann, G. Schramm, U. Schumacher, and the ASDEX Upgrade team,

Rev. Sci. Instrum. **67** (5), May 1996

C-/O- Monitor for ASDEX-U

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for C VI, $\lambda=33.74 \text{ \AA}$:

Crystal:

PbSt

$2d=100.4 \text{ \AA}$;

Filters:

$1 \text{ \mu m}$ polyamid,
 $20 \text{ \AA}$ Al.

for O VIII, $\lambda= 18.97 \text{ \AA}$:

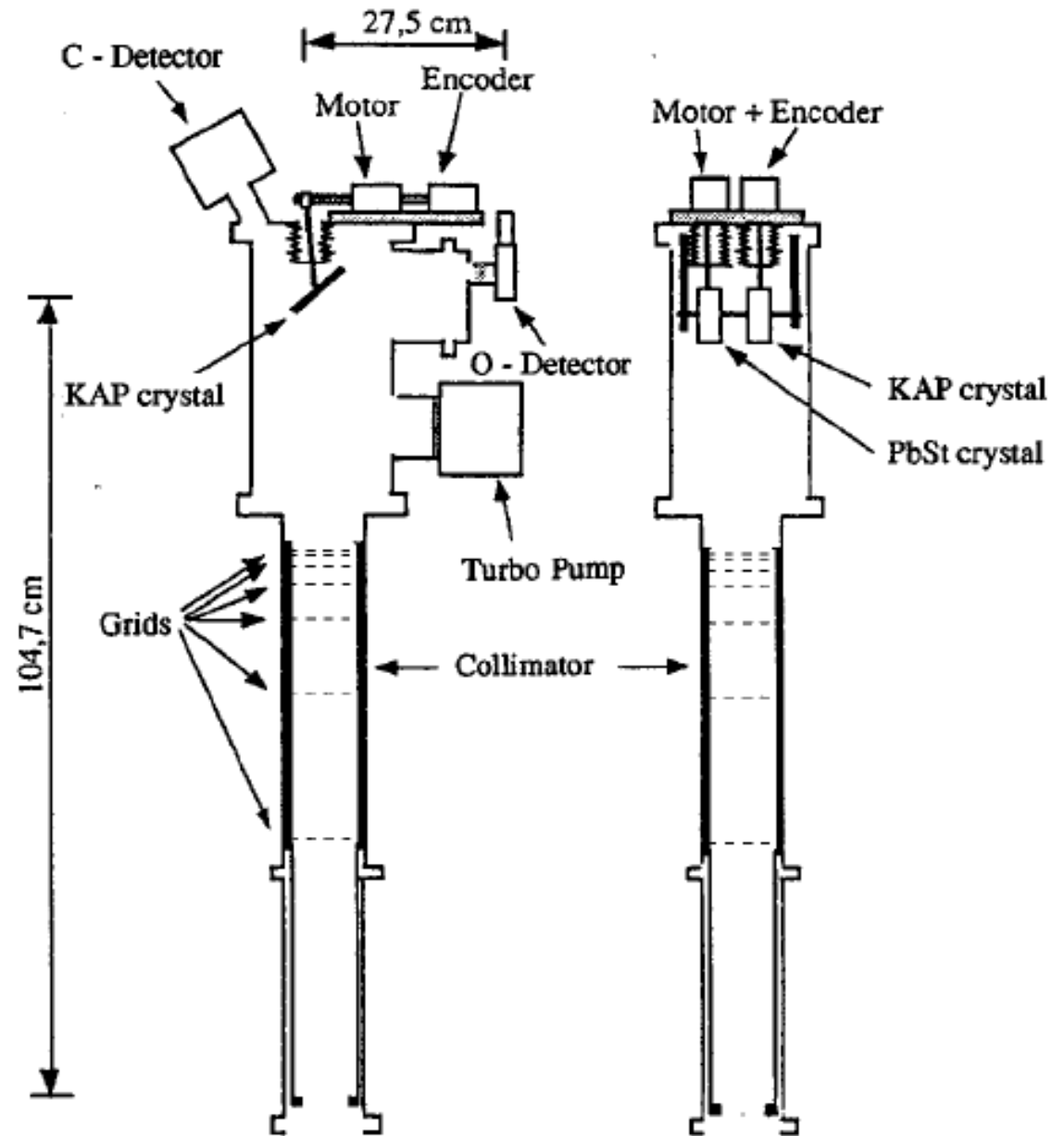
Crystal:

KAP

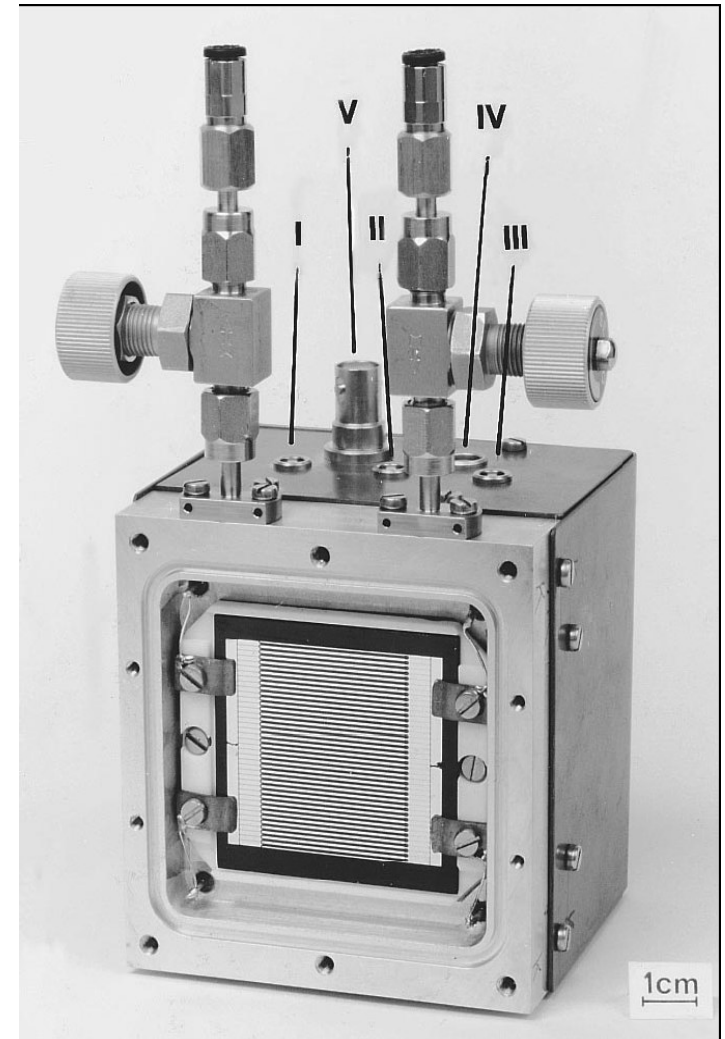
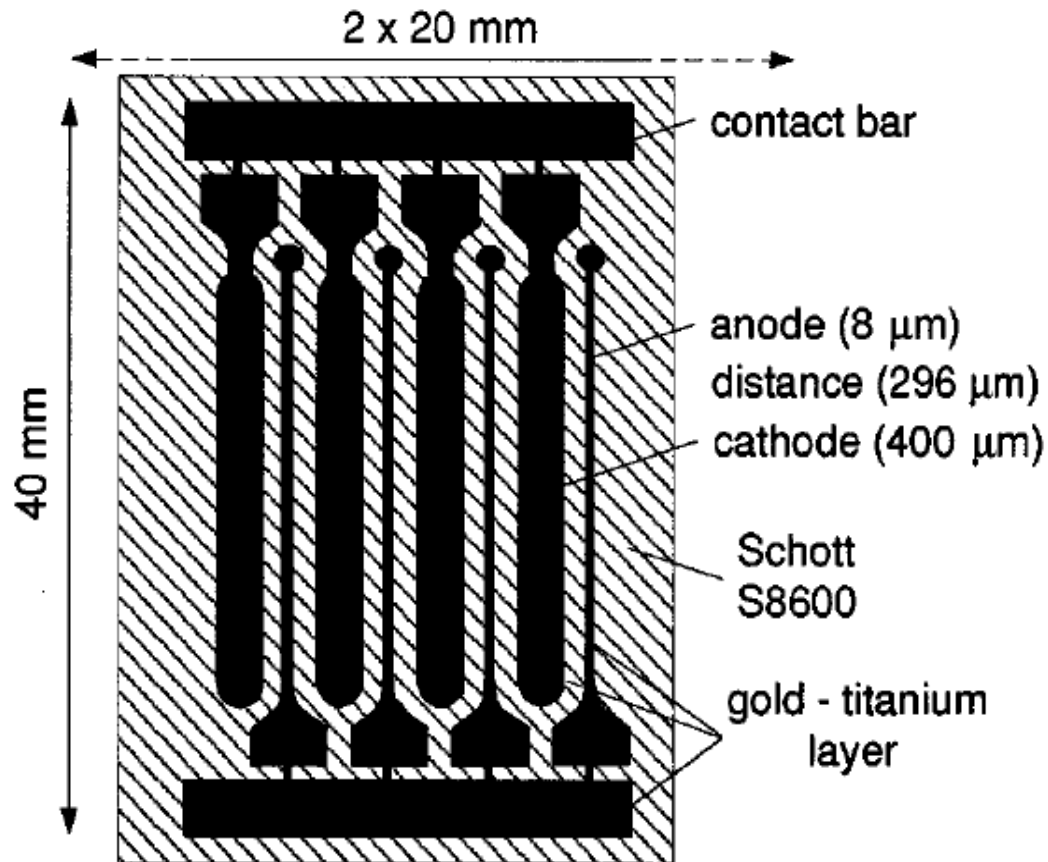
$2d=26.58 \text{ \AA}$;

Filters:

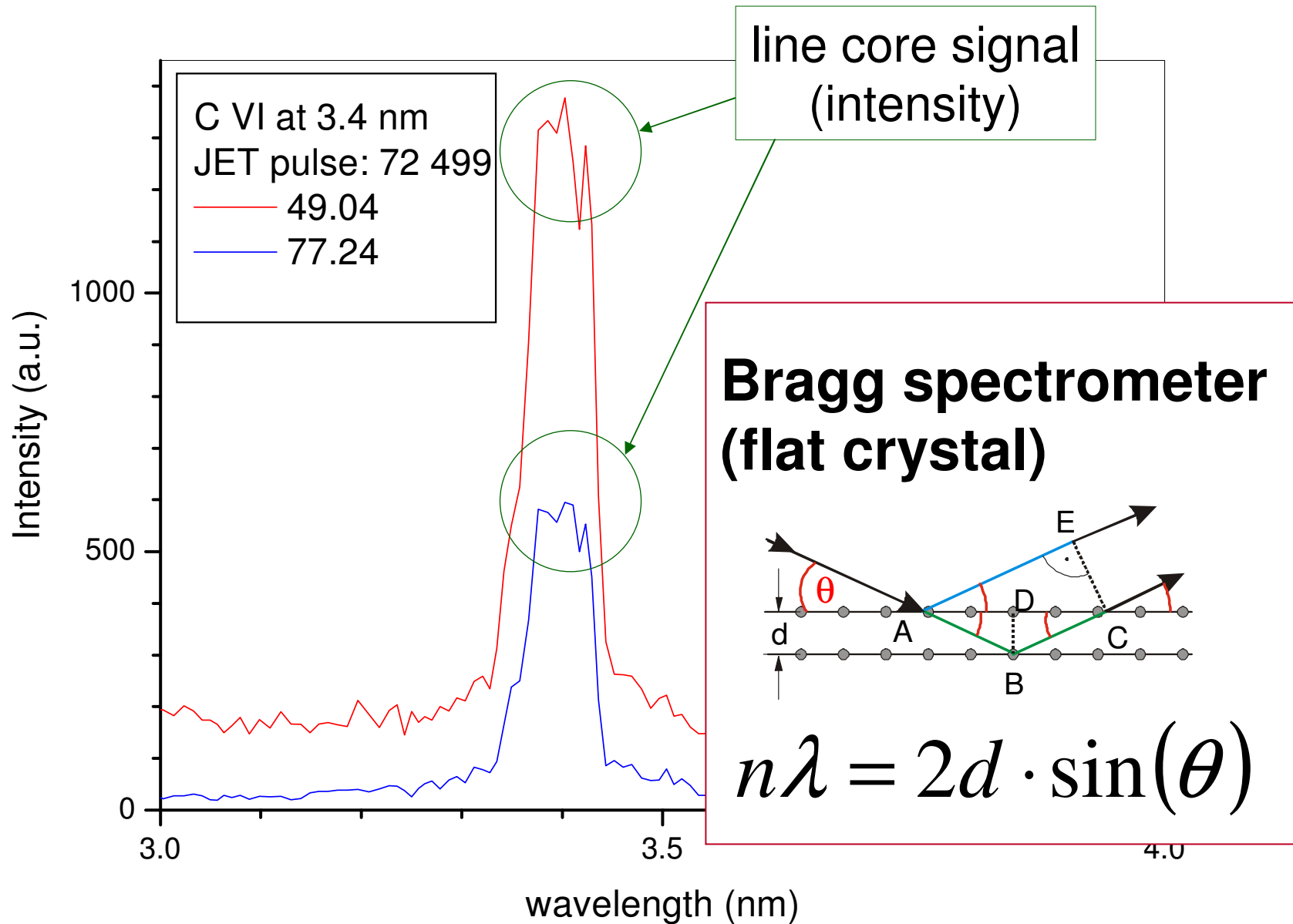
$2 \text{ \mu m}$ mylar,
 $100 \text{ \AA}$ Al.



Proportional counter



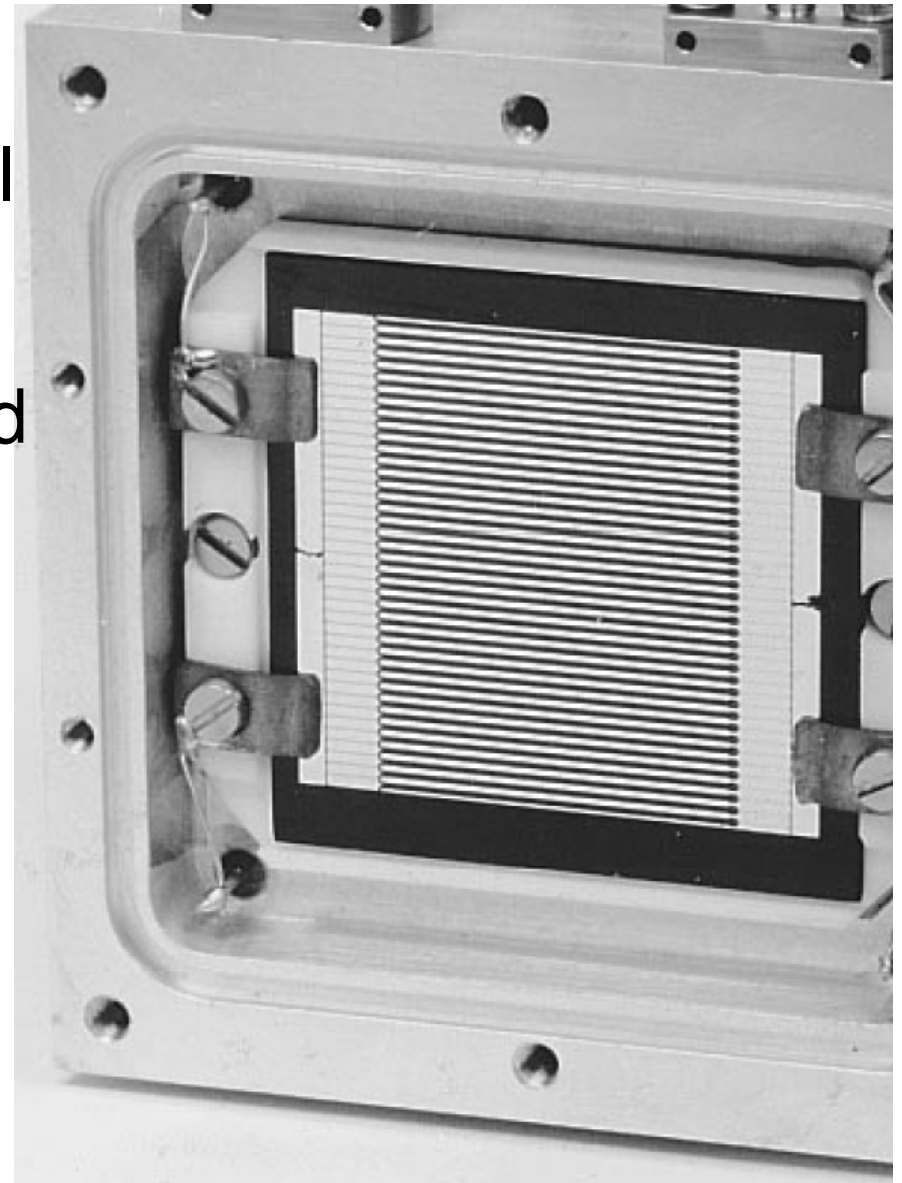
line intensity = area under profile curve



Main disadvantage of this construction:

As it is flat crystal Bragg-spectrometer background level is **estimated** by the count rate arising from the high energy gammas and other background radiation.

(Each proportional counter consists of two independent active areas of identical $30 \times 20 \text{ mm}^2$ size. One of them is attached directly to the spectrometer using thin windows, whereas the second one has no window.)



Sample of results of by C-/O- Monitor for ASDEX-U

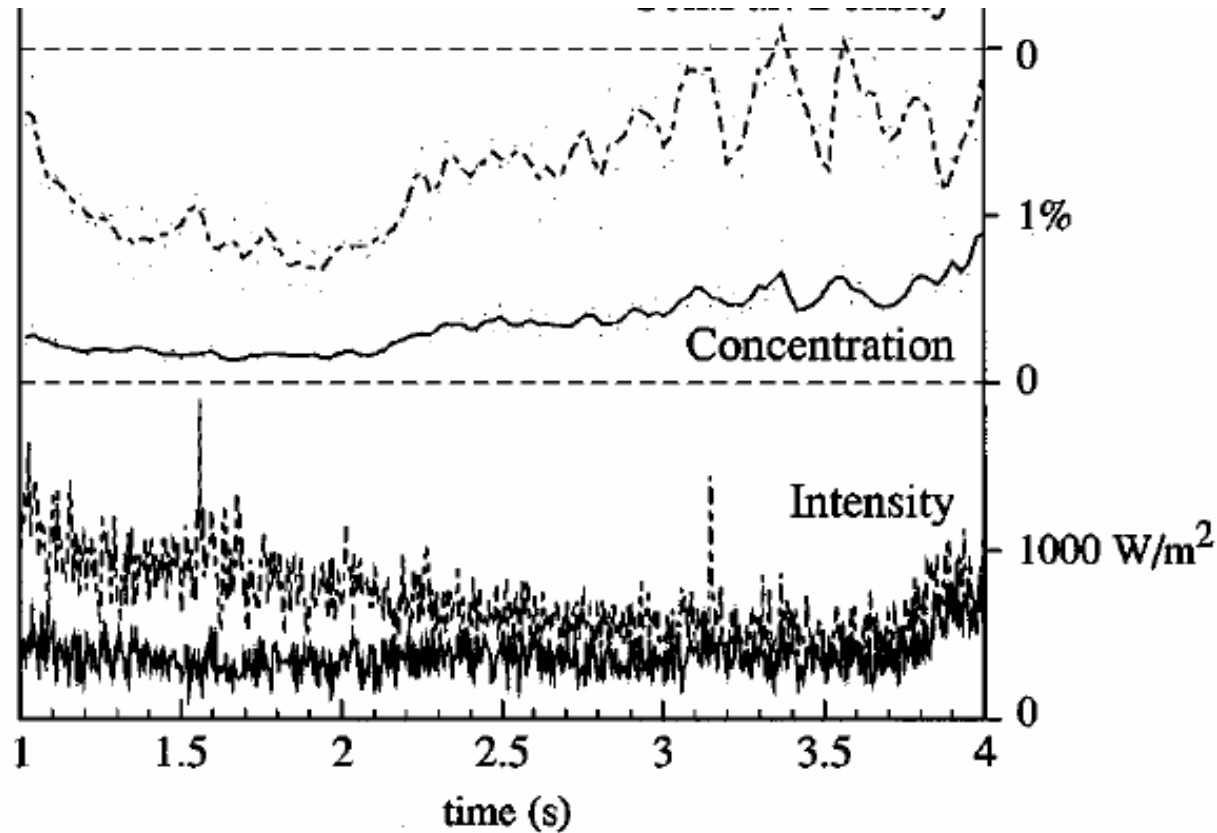


FIG. 7. Temporal behavior of the C VI and O VIII line intensities (multiplied by the factor 4π), together with the deduced concentrations and control parameters (O: solid lines; C: dashed lines) from the ASDEX Upgrade standard discharge No. 4611 ($\overline{n_e} = 3 \times 10^{19}/\text{m}^3$, $I_p = 600$ kA).

Sample of results of by C-/O- Monitor for ASDEX-U

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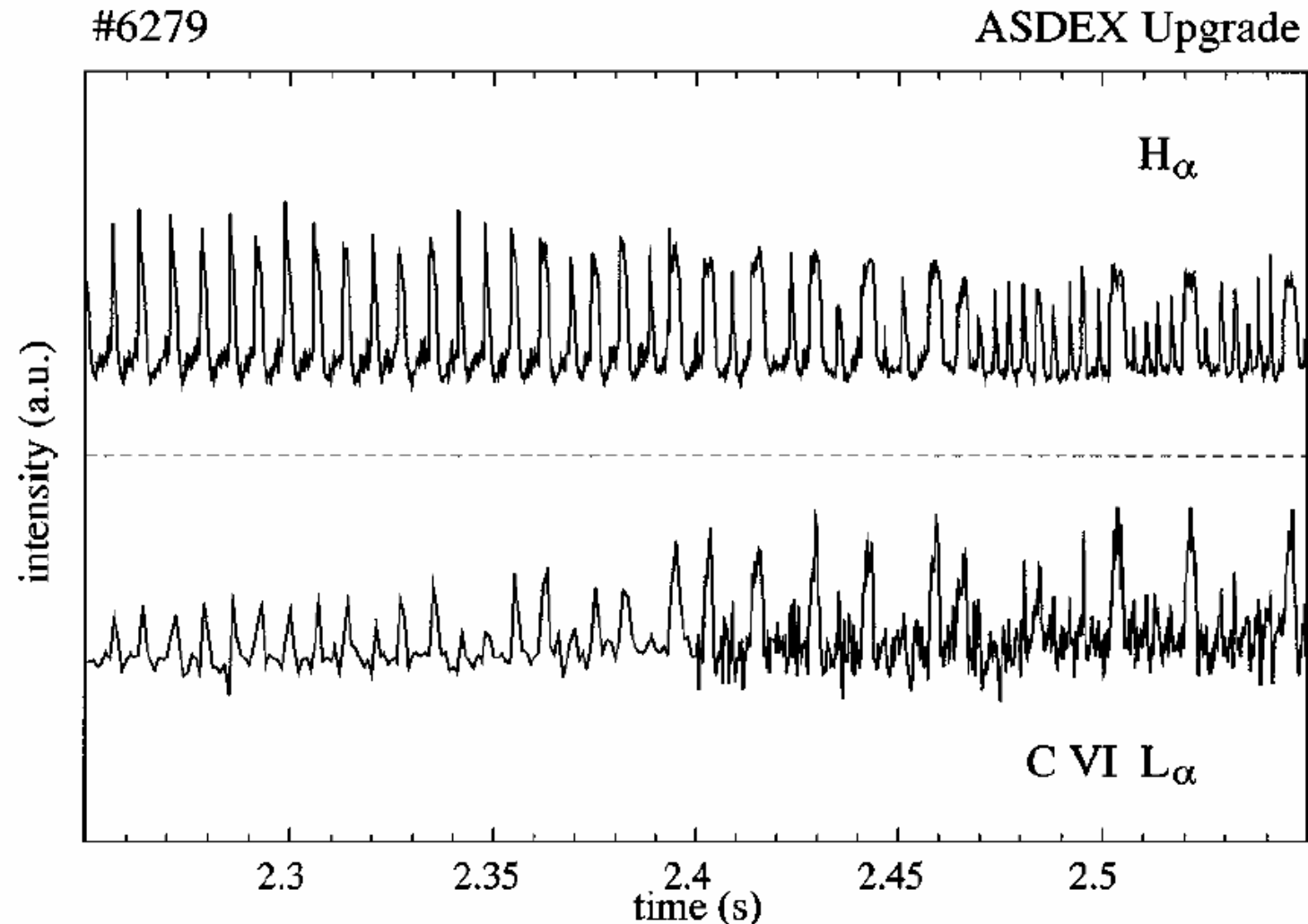


FIG. 9. C VI Lyman- α intensity as a function of time (lower curve) showing ELM activity as the H_{α} radiation measured in the divertor (upper curve).

C-/O- Monitor for Wendelstein 7-X

C- O- Monitor for Wendelstein 7-X

- will be a dedicated soft X-ray spectrometer;
- working with time resolution of at least 0.5 ms;
- it consists of four independent channels, fixed at wavelengths corresponding to Lyman- α lines of hydrogen-like ions of:
 - boron (at 4.9 nm; 253 eV),
 - carbon (at 3.4 nm; 365 eV),
 - nitrogen (at 2.5 nm; 496 eV),
 - oxygen (at 1.9 nm; 653 eV).

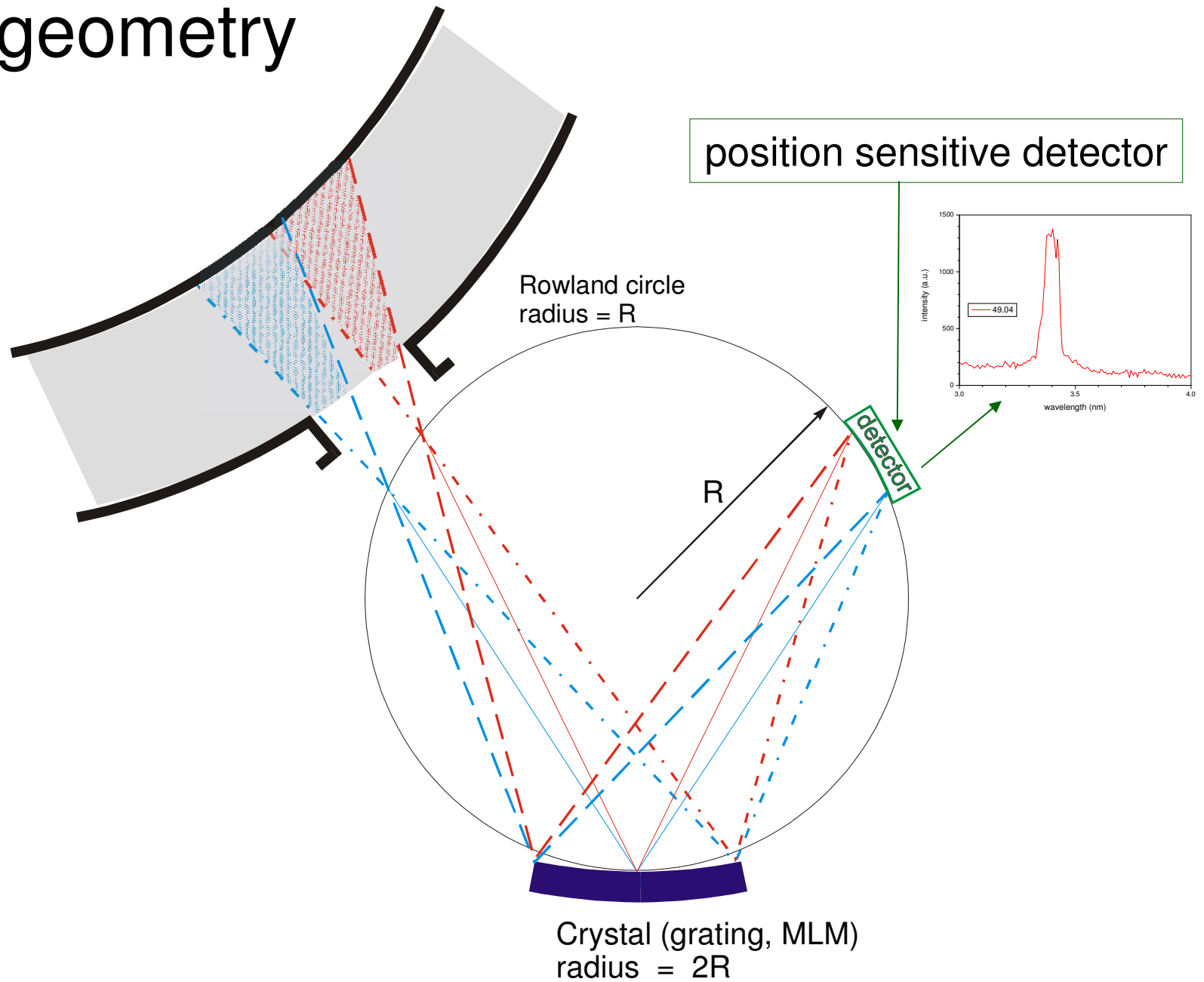
C- O- Monitor for W7-X

it consists of four independent channels, but

divided in two pairs, closed in common vacuum chambers:

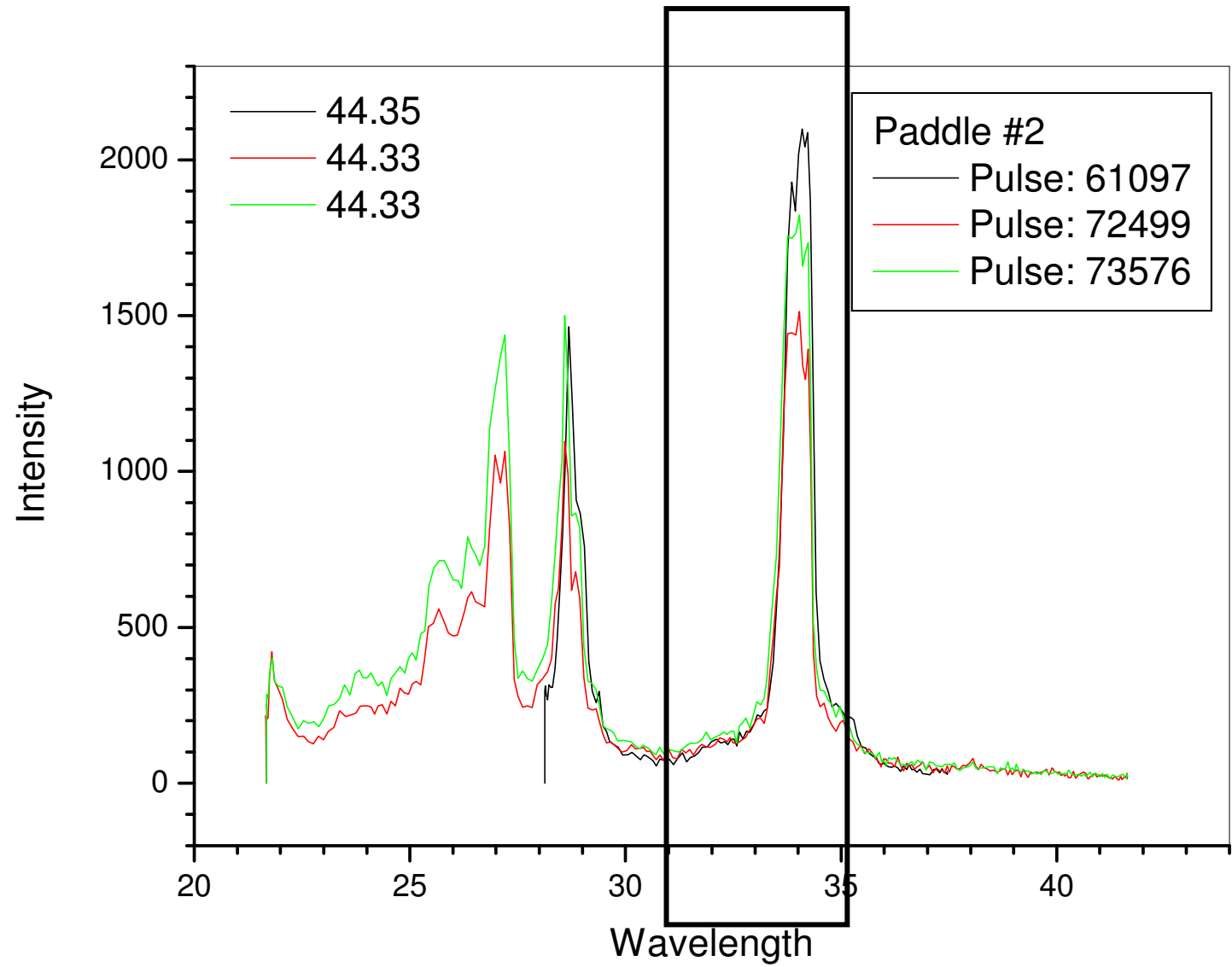
- 1) carbon and oxygen,
- 2) boron and nitrogen.

Johan geometry

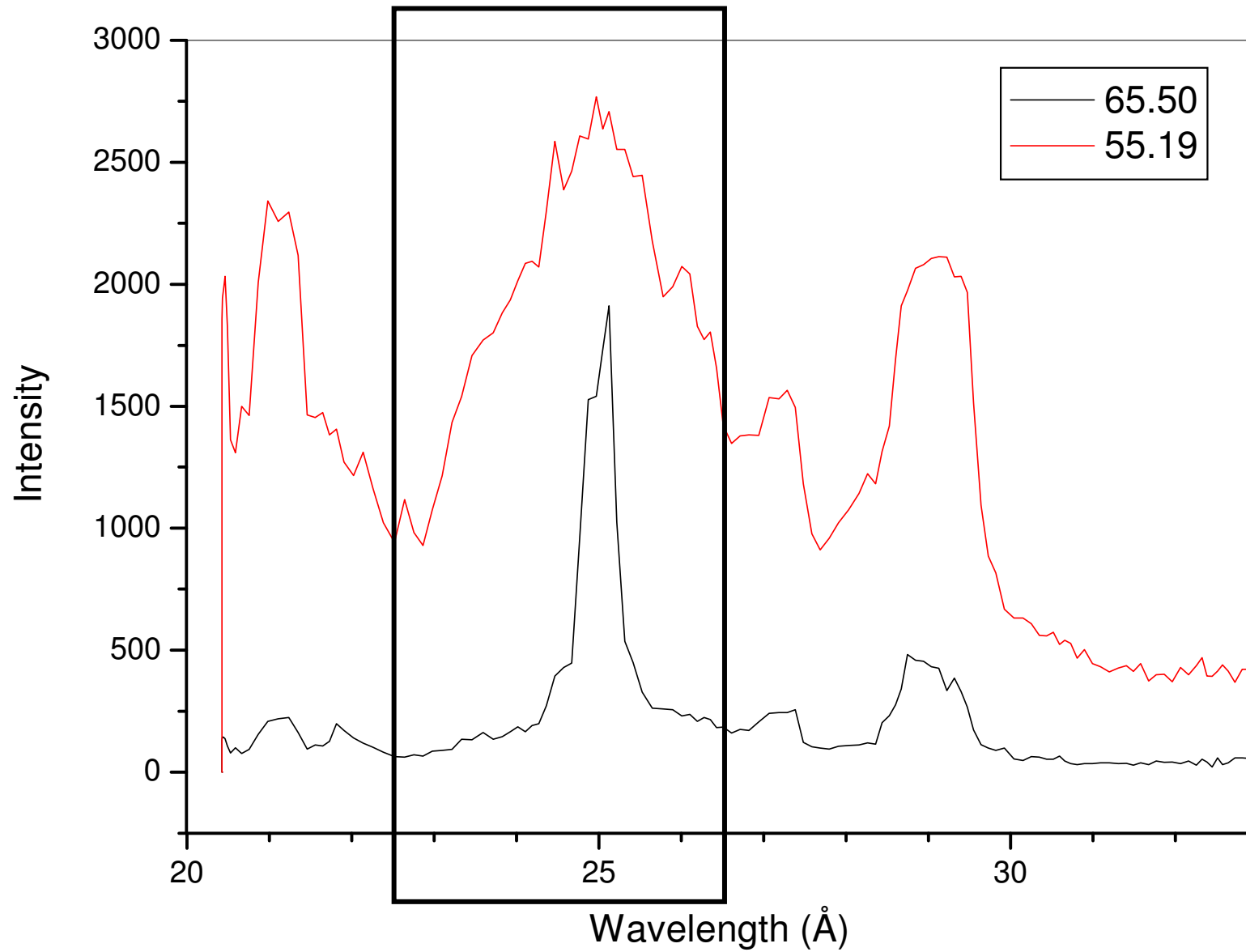


Crystal/MLM cylindrically bend

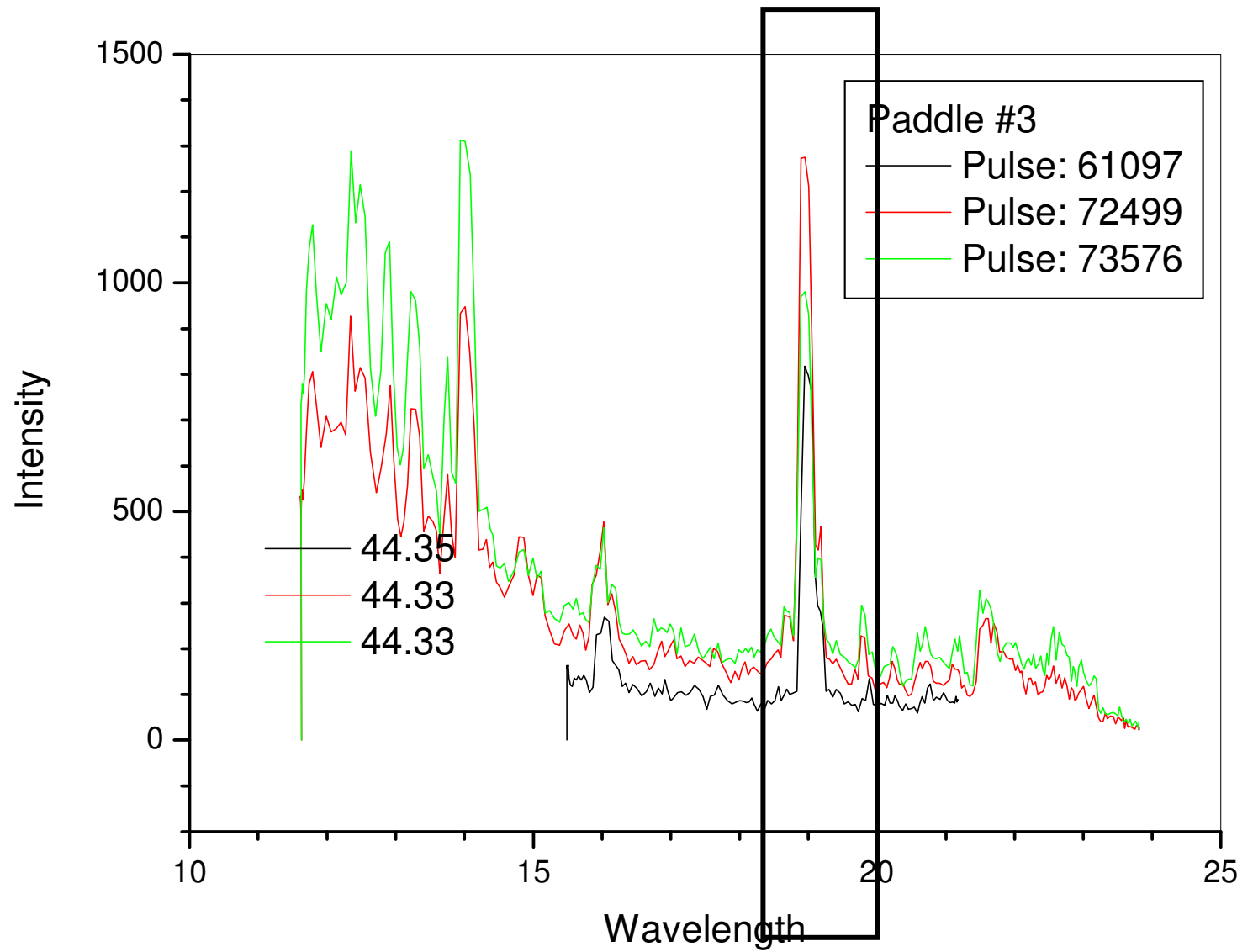
C VI spectrum

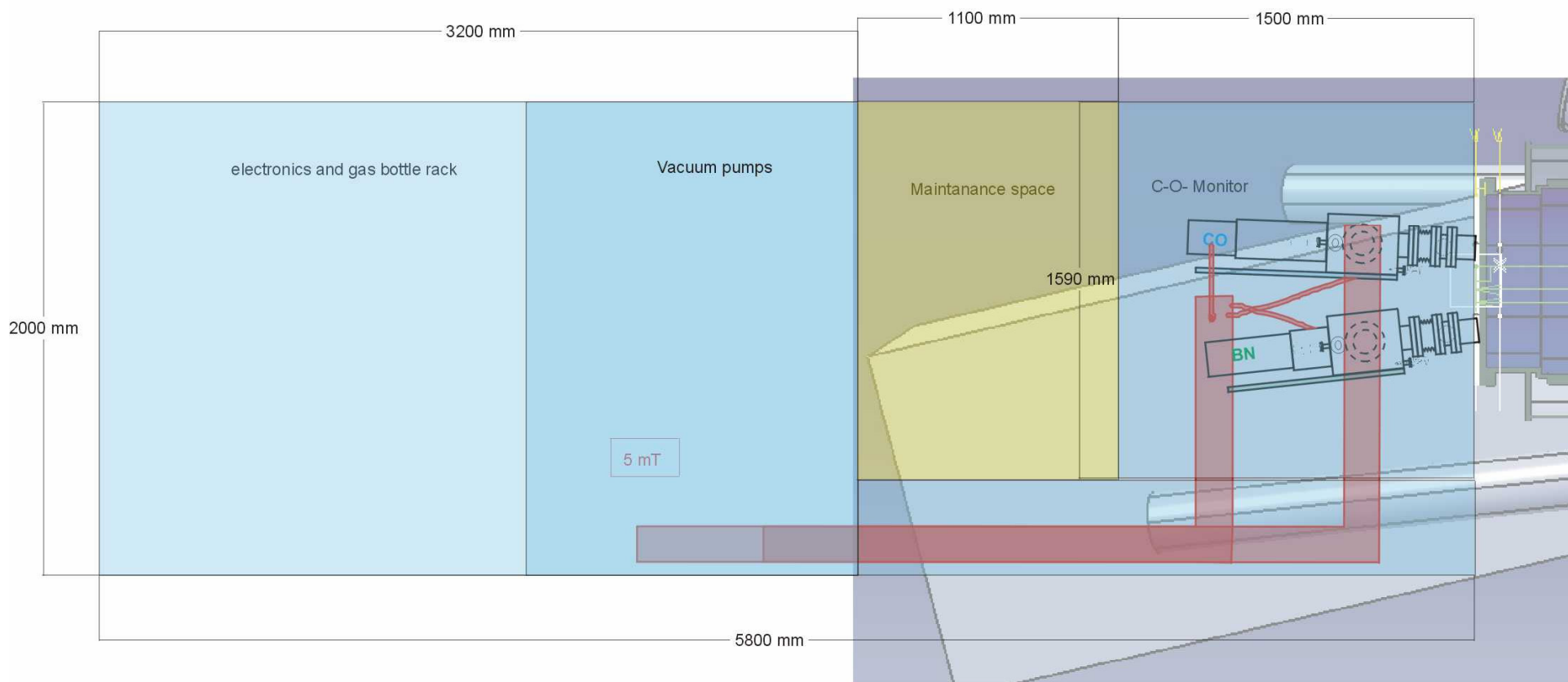


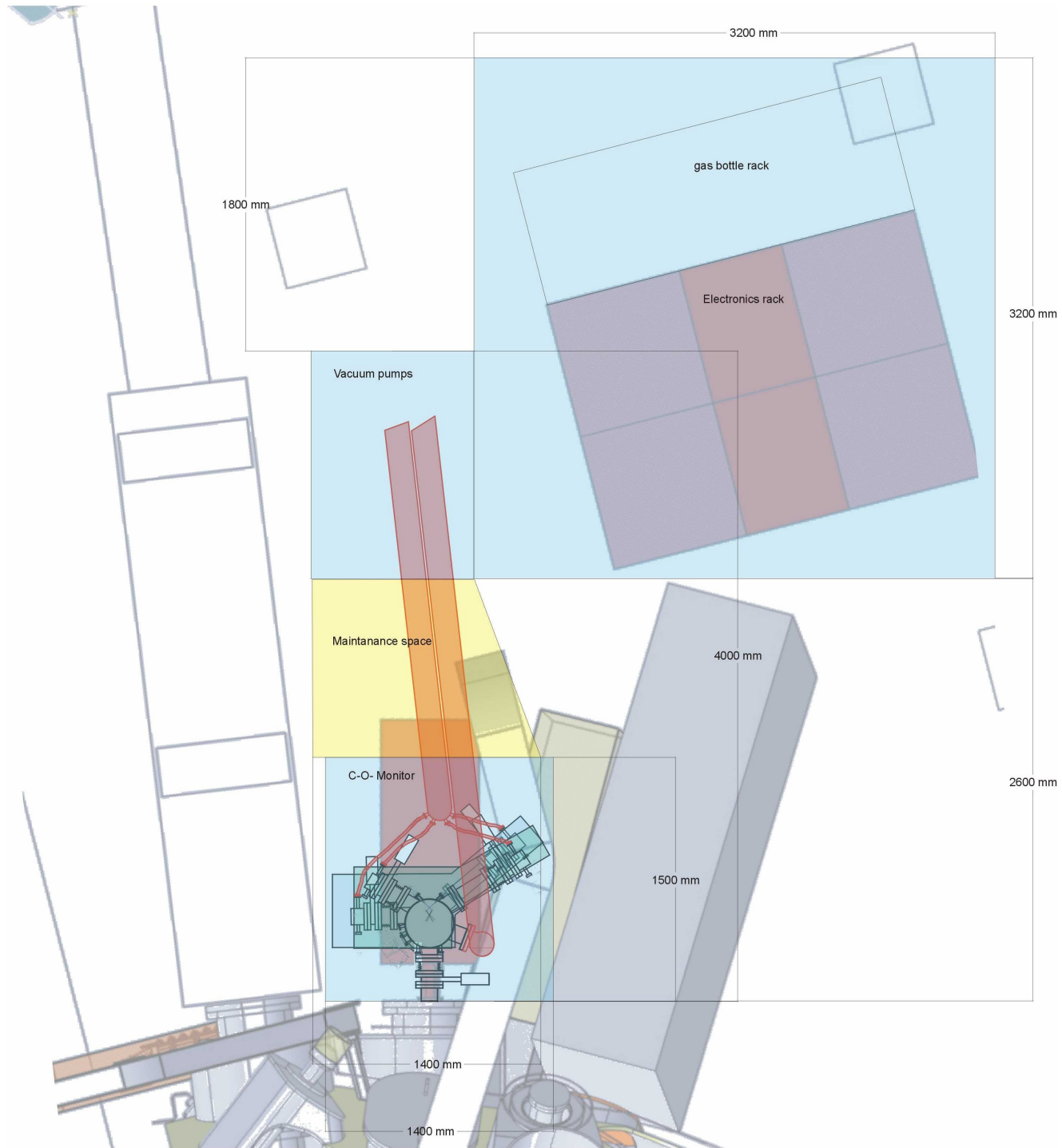
N VII spectrum



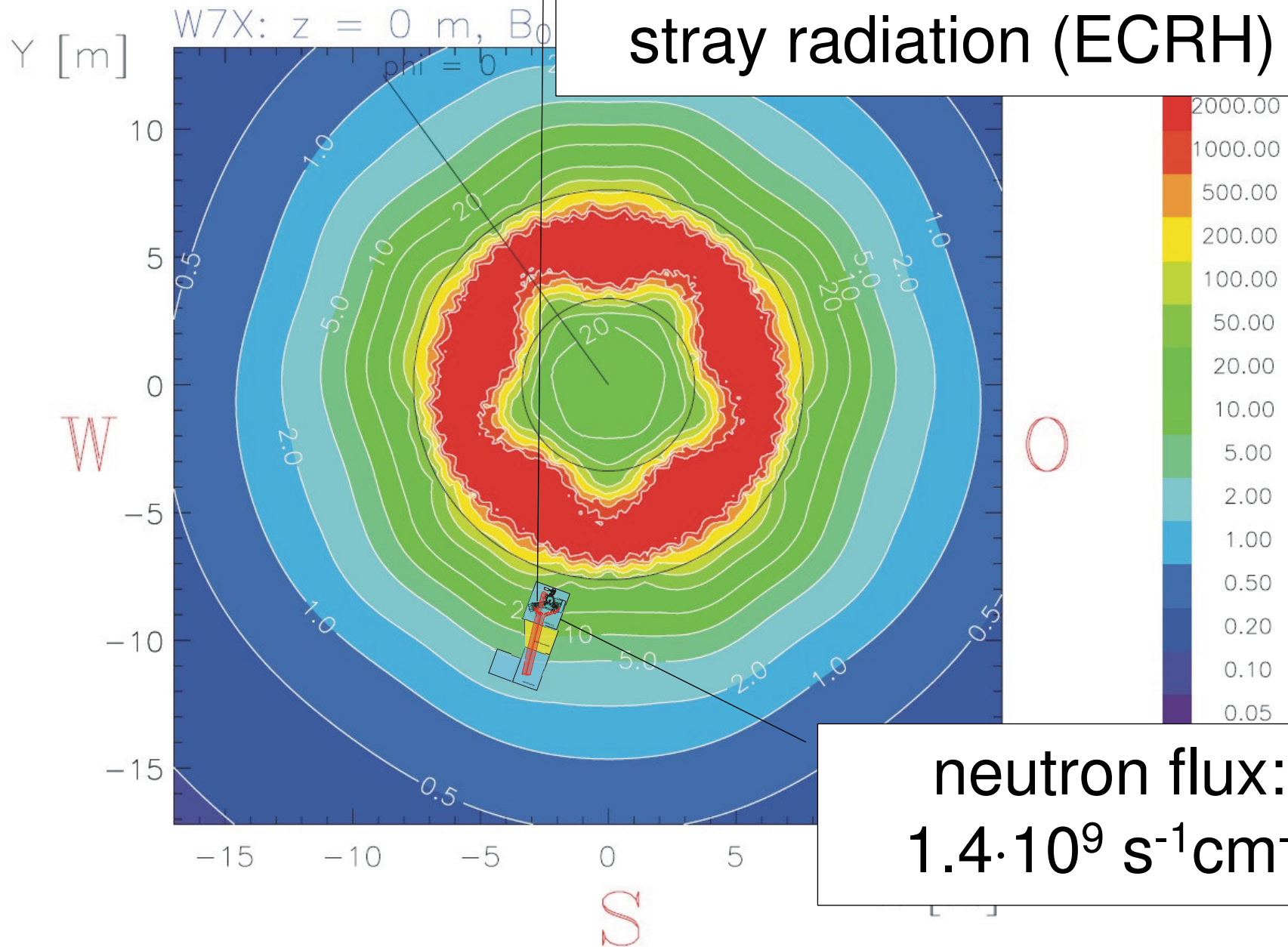
O VIII spectrum





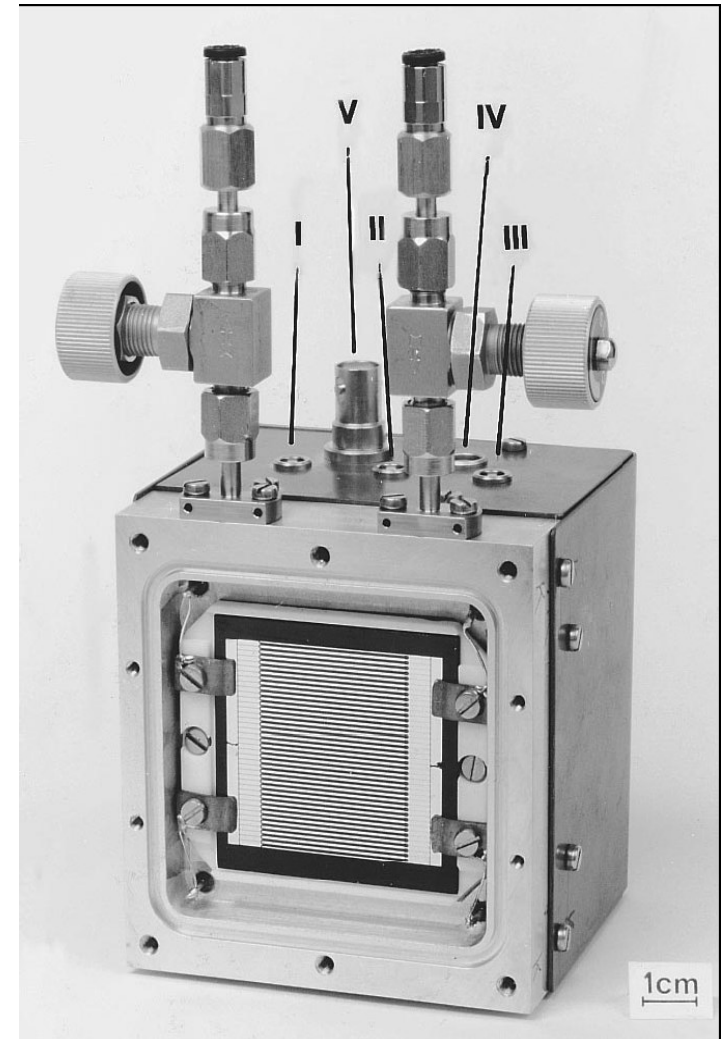
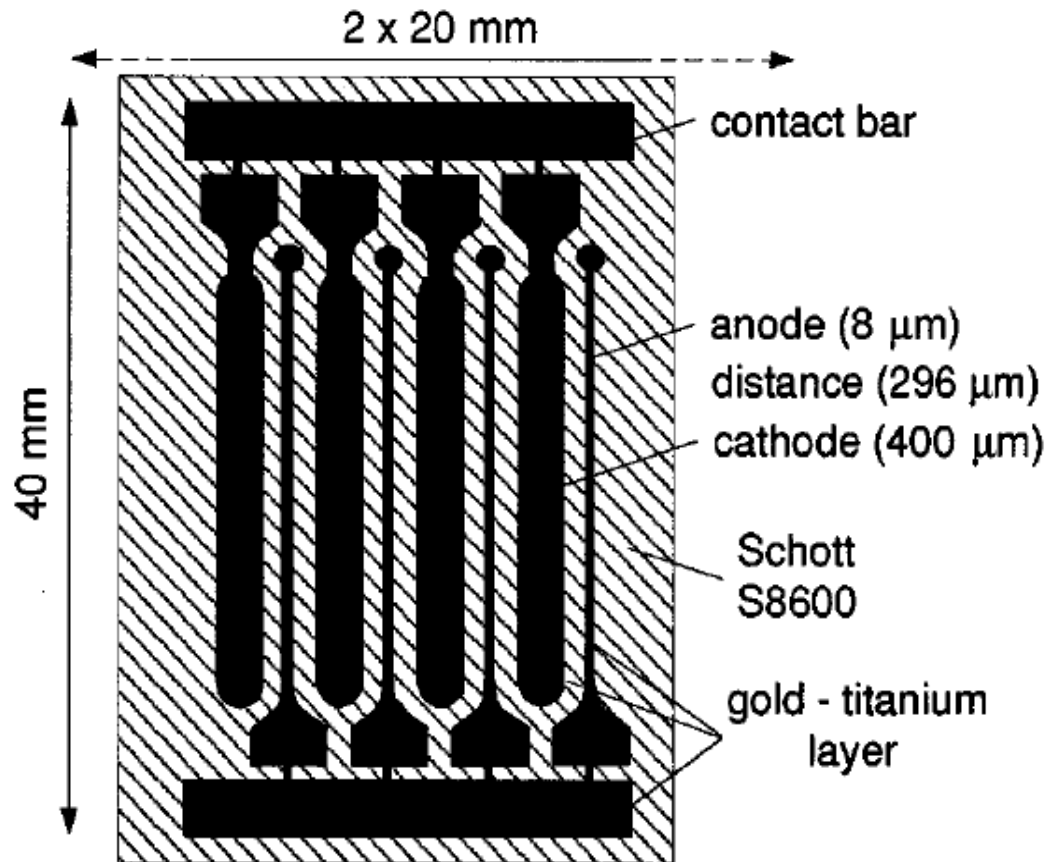


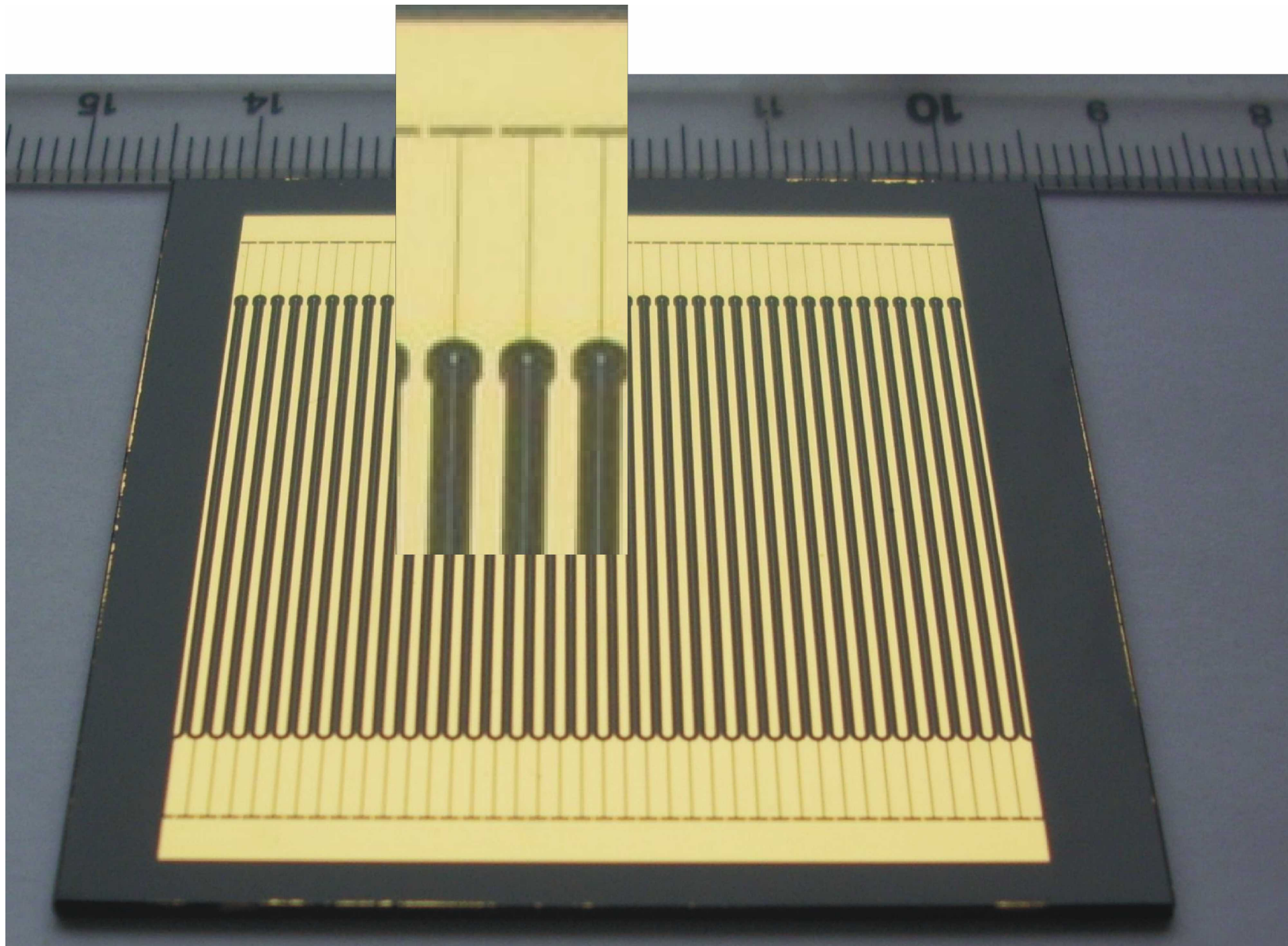
Microwave (140 GHz) stray radiation (ECRH)

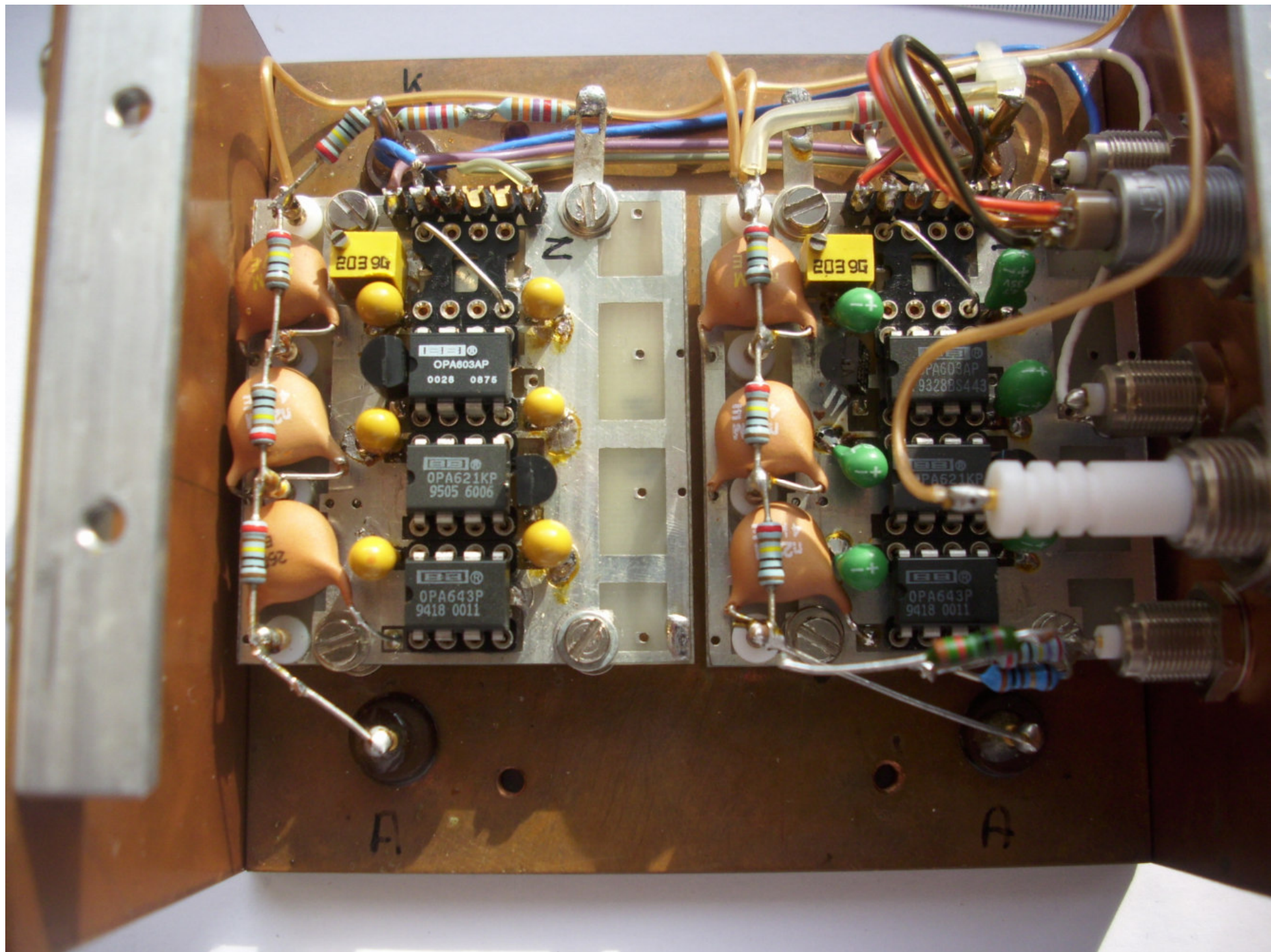


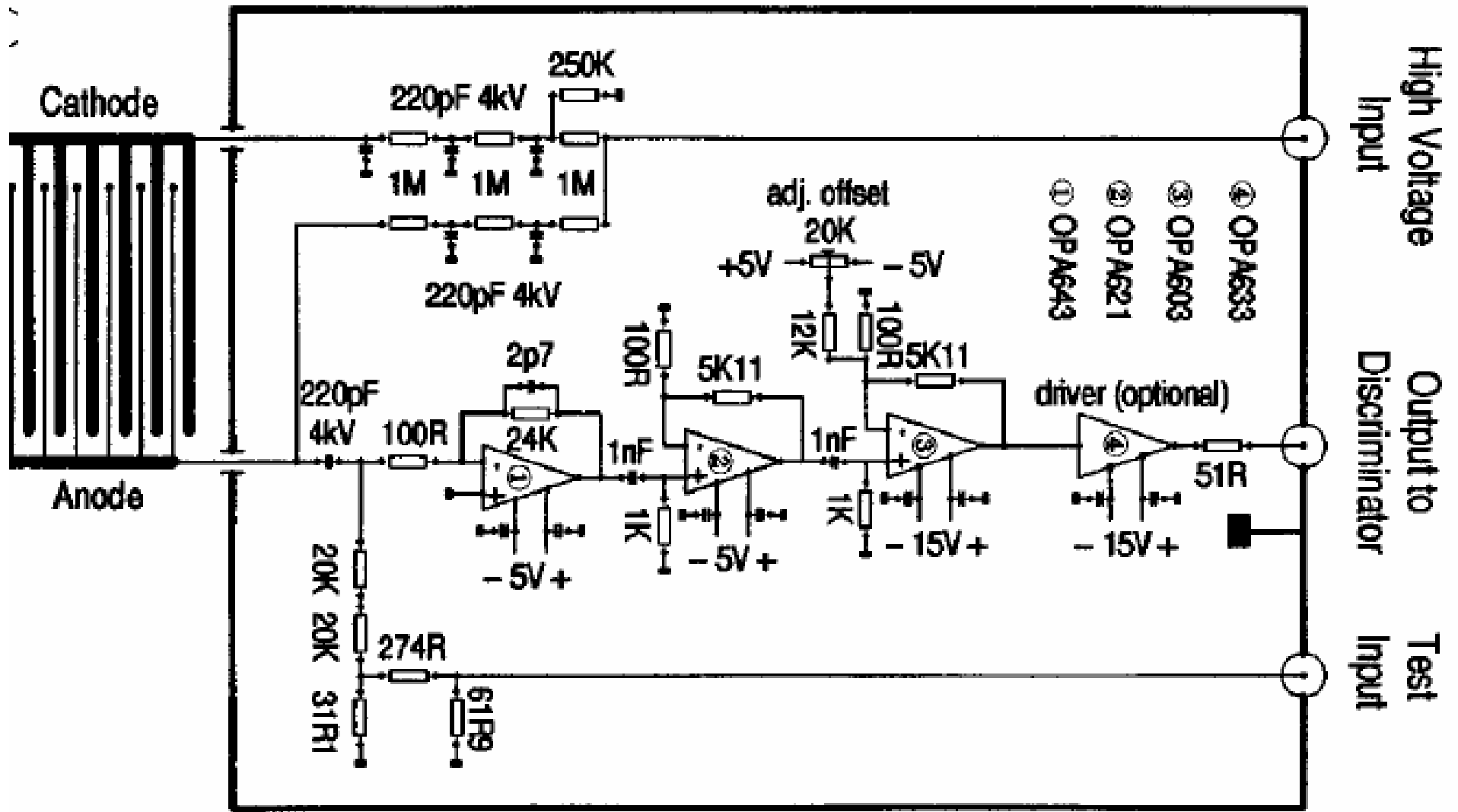
Proportional counter for C-/O- Monitor

Proportional counter



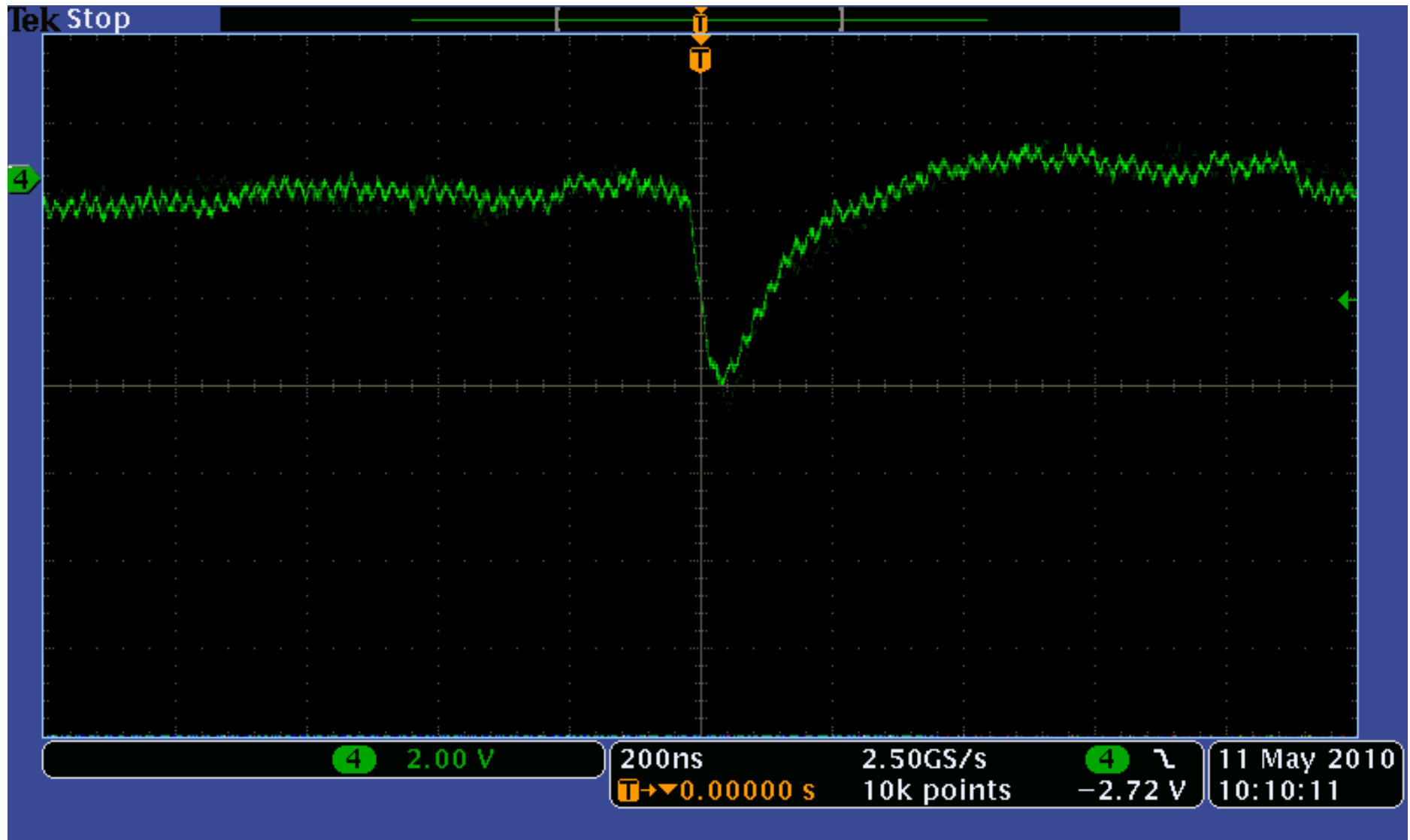




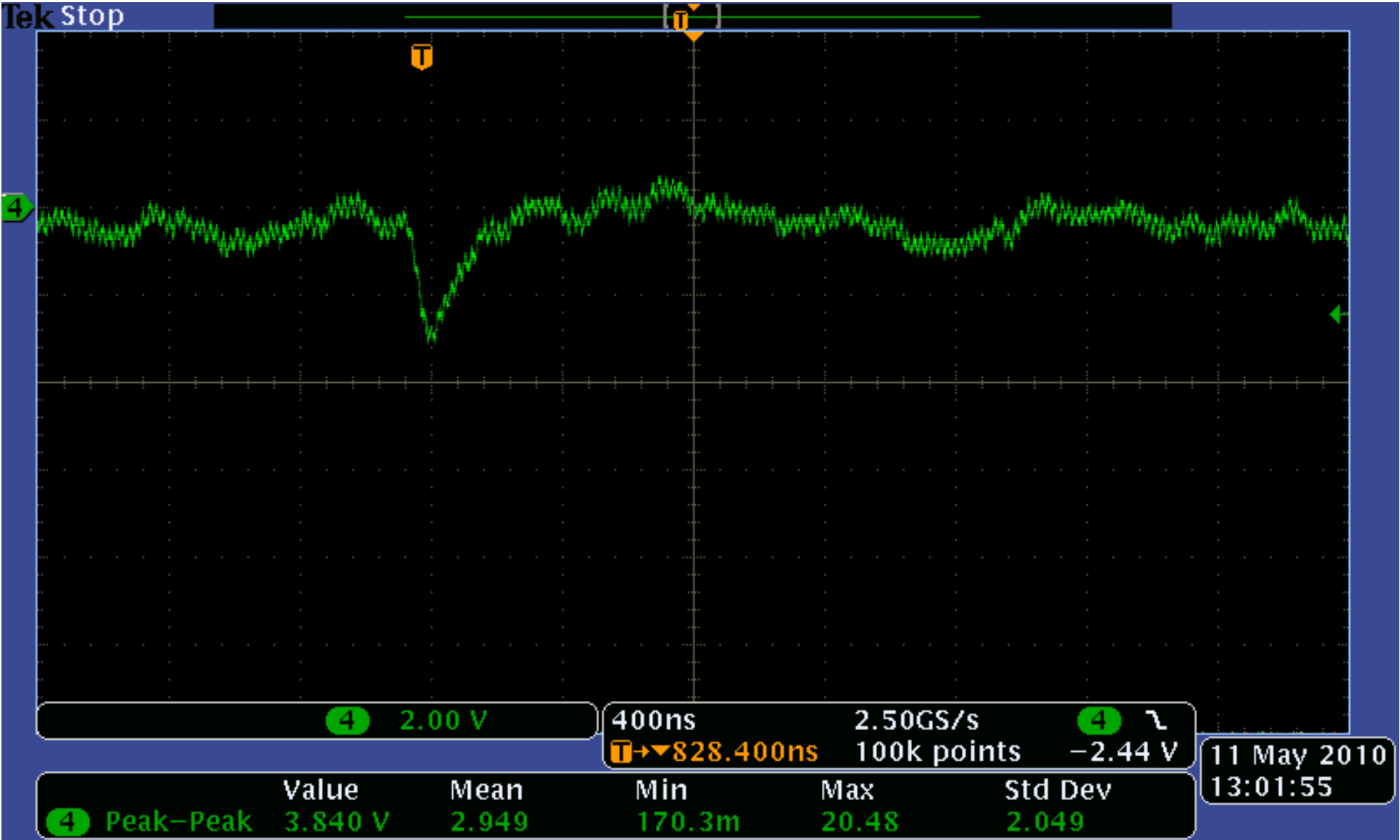


Al K α signal (1 487 eV)

48



Cu L β signal (930 eV)



Proportional counter:

**for ASDEX-U
(what we have)**

**for WENDELSTEIN 7-X
(what we want)**

Readout of 2 channels

Readout of 40 channels

Measurement of line
core and indirect
estimation of continuum

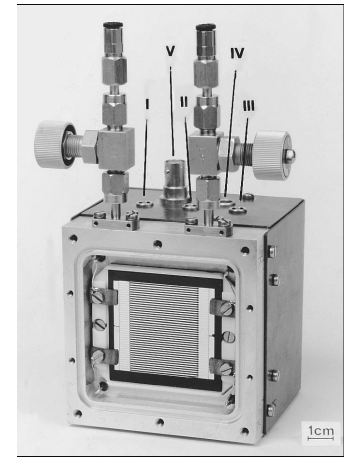
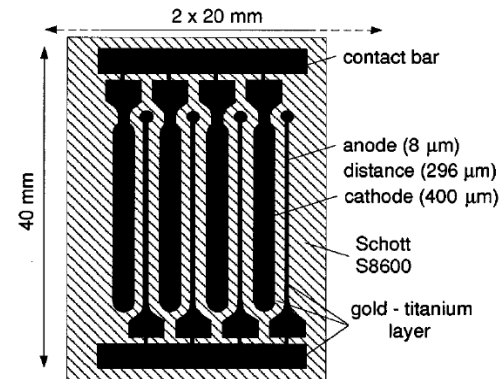
Measurement of line
core as well as
continuum signal

High temporal resolution

line intensity with time
resolution < 0.5 ms

Thank you
for your attention

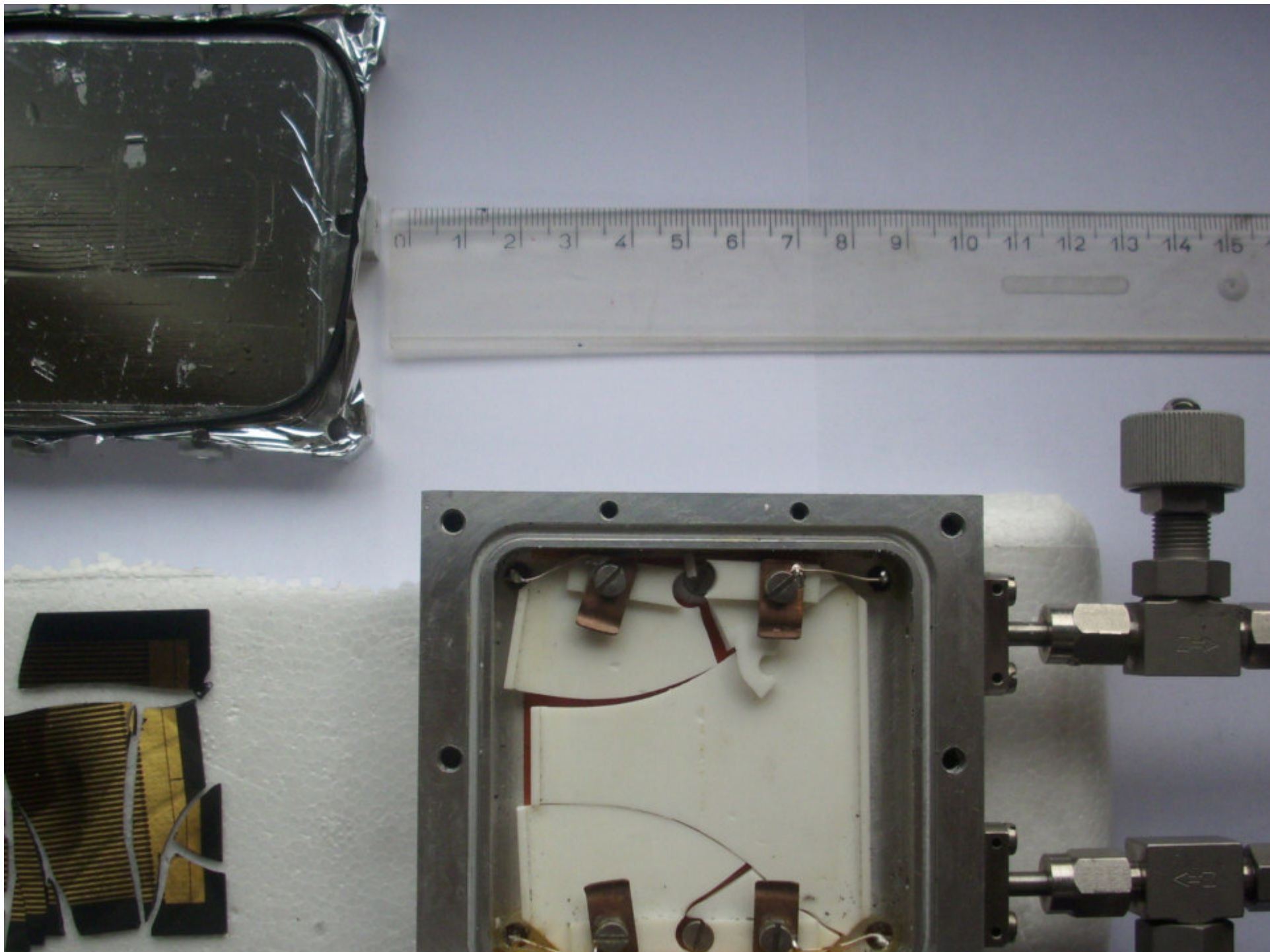
Proportional counter modification

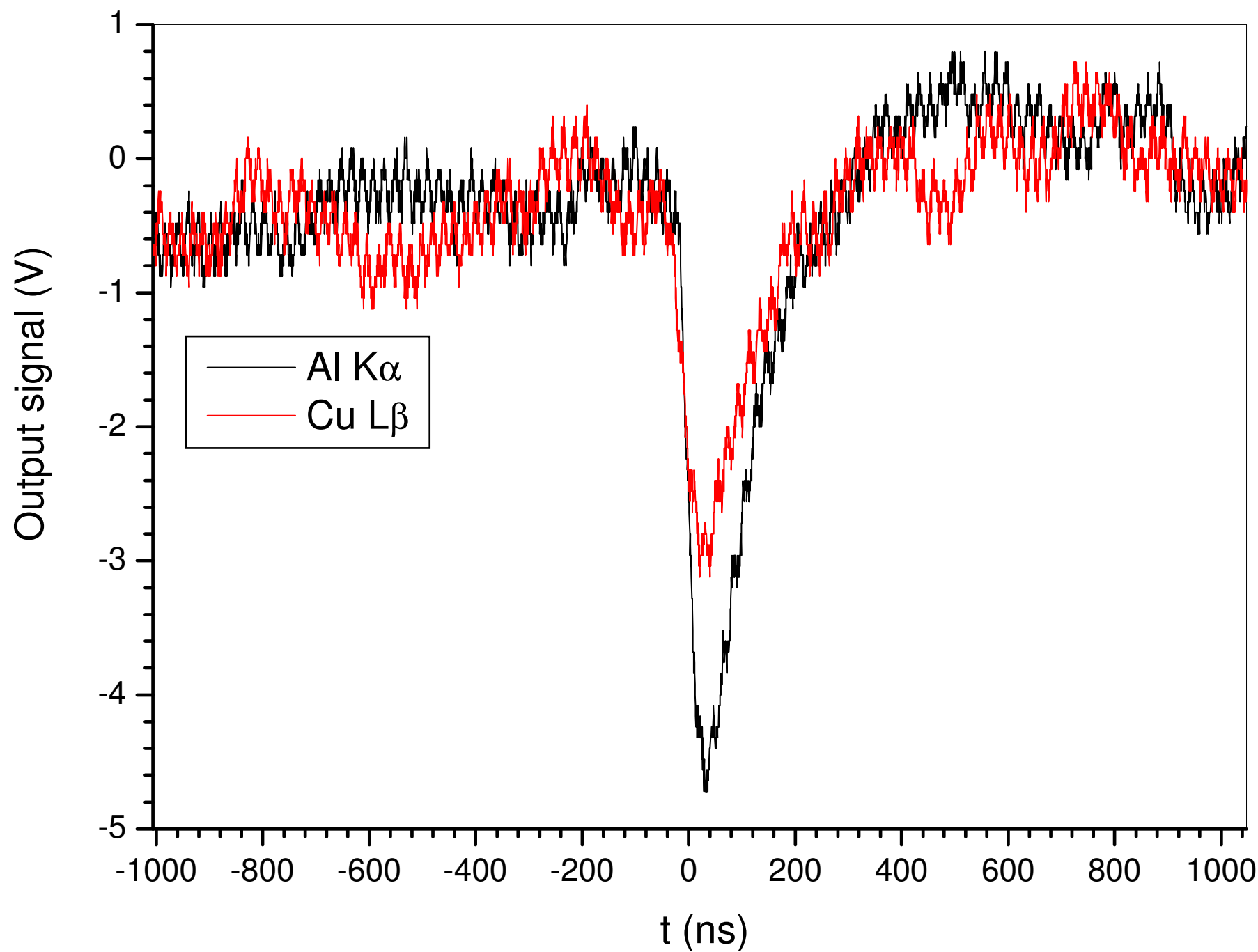


Readout of 40 separate channels

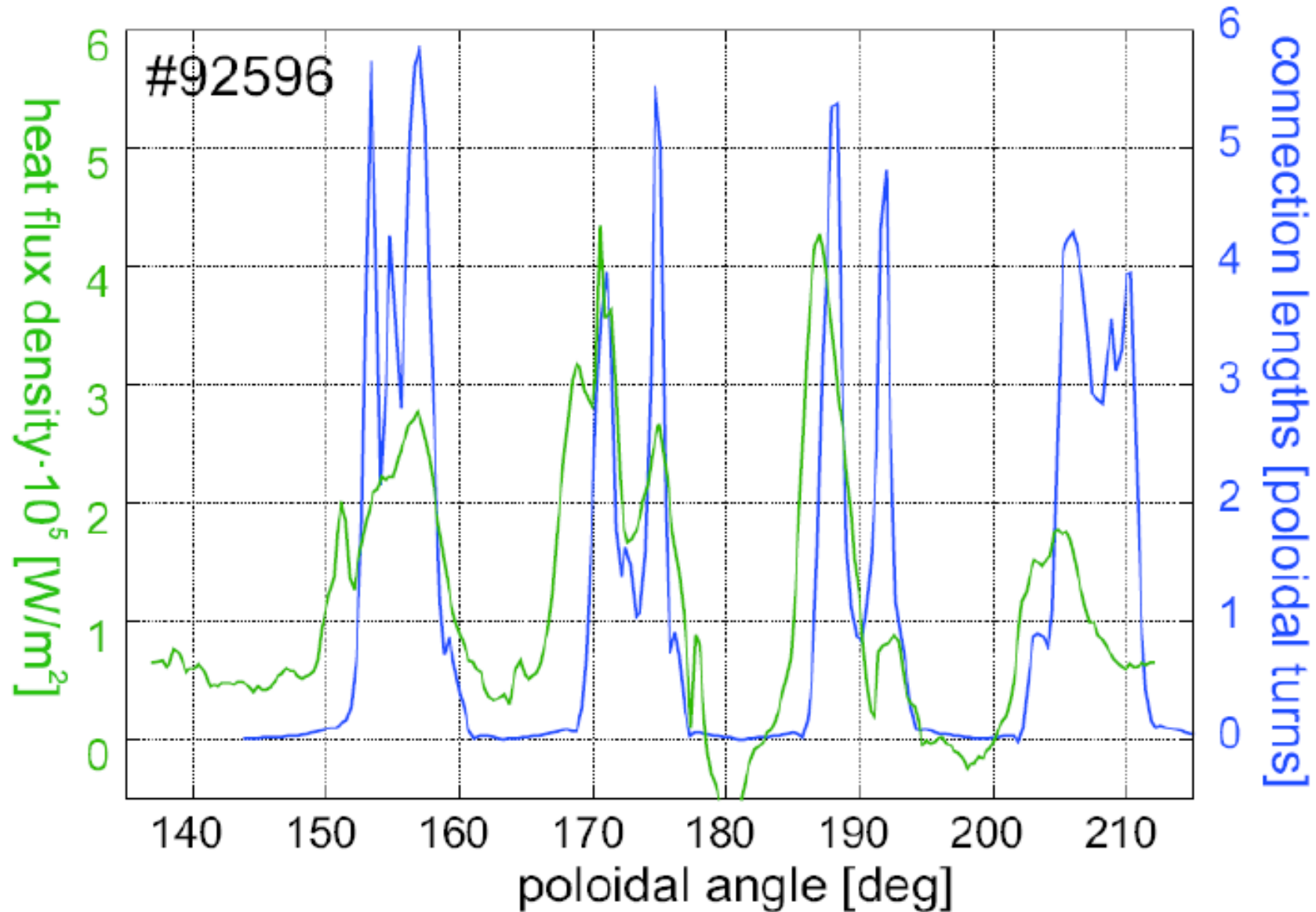
Measurement of line core as well as continuum signal

Determination of line intensity with time resolution < 0.5 ms



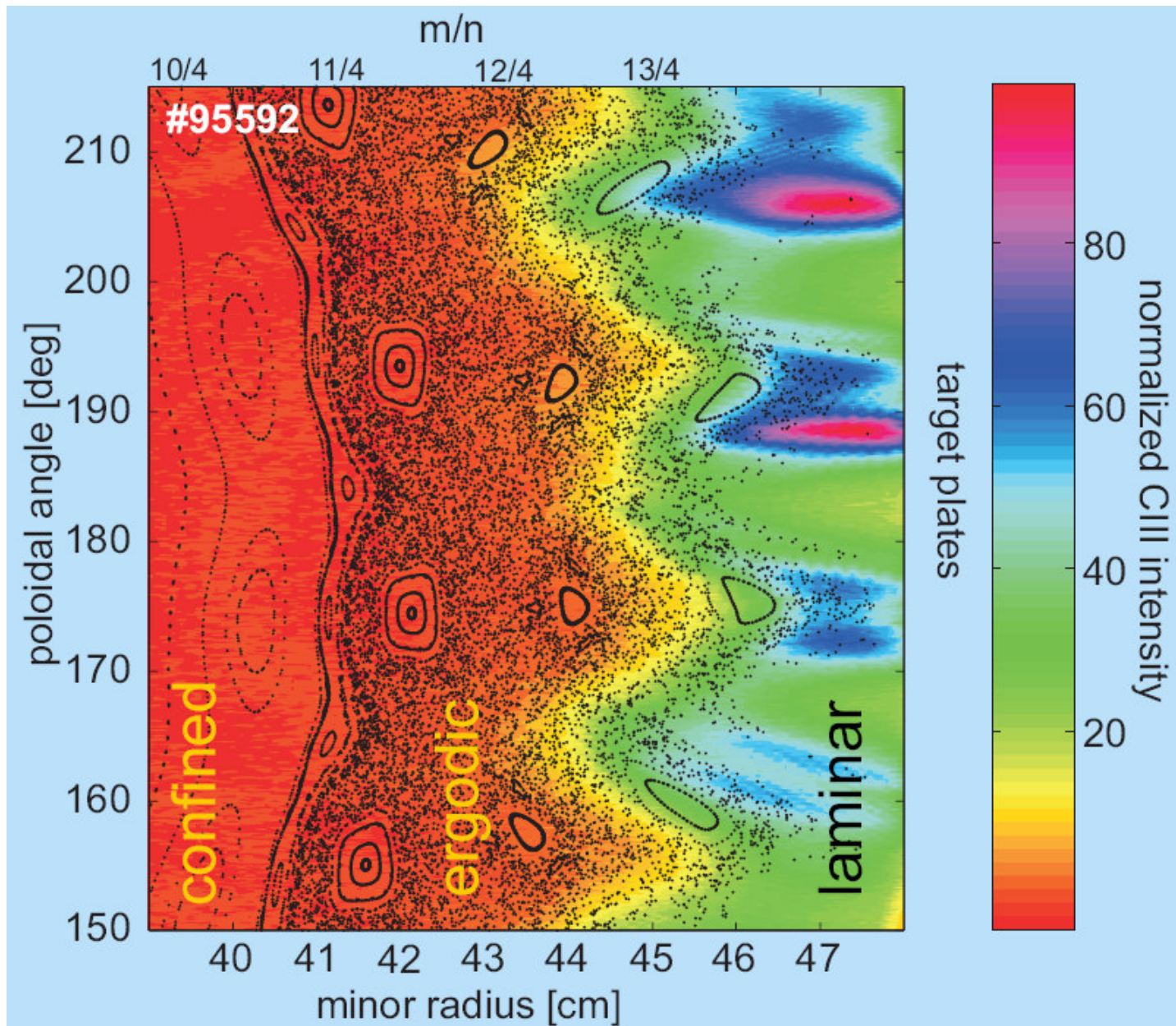


ELMs (Edge Localized Modes)



ELMs – Edge Localized Modes

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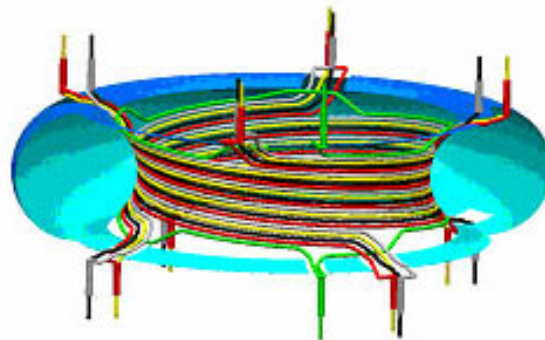
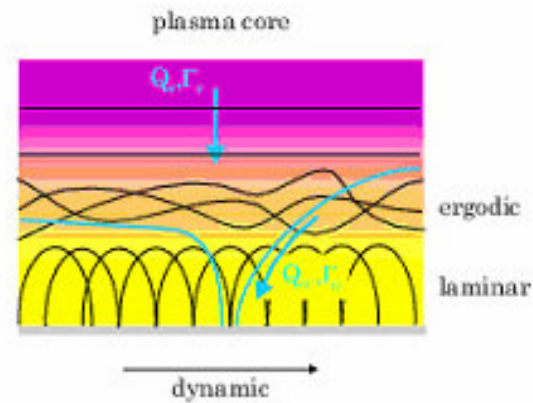


DED – Dynamic Ergodic Divertor

"conventional" divertor



dynamic ergodic divertor



H-mode

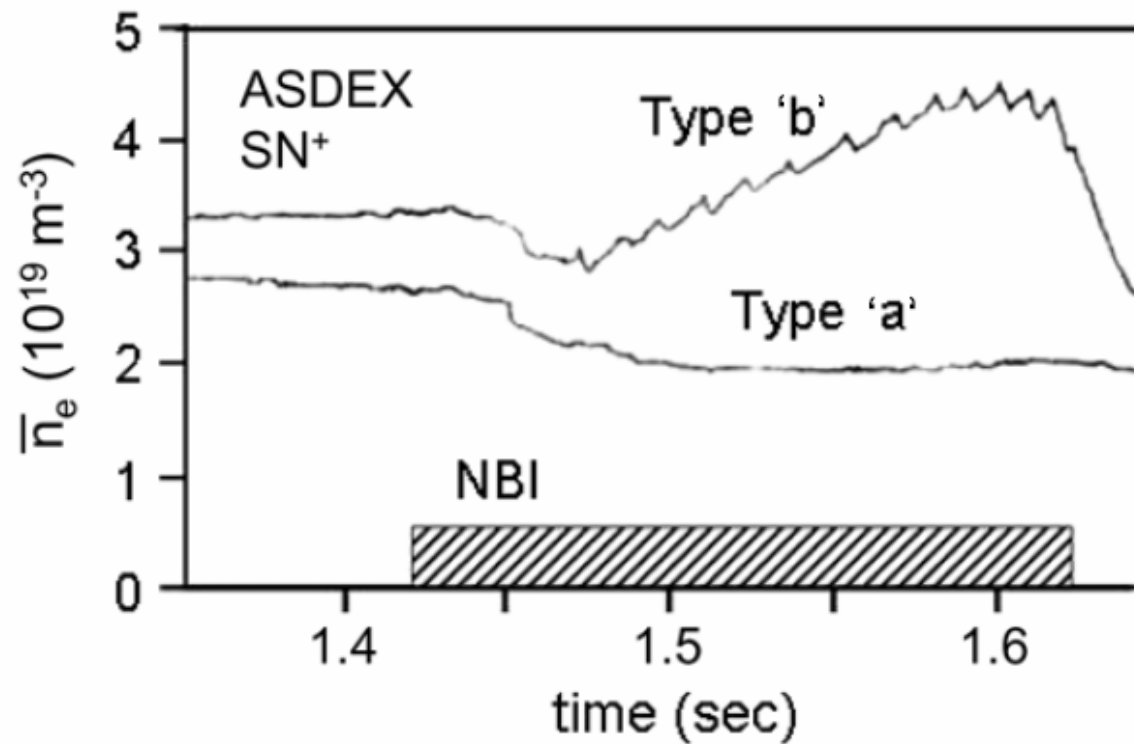
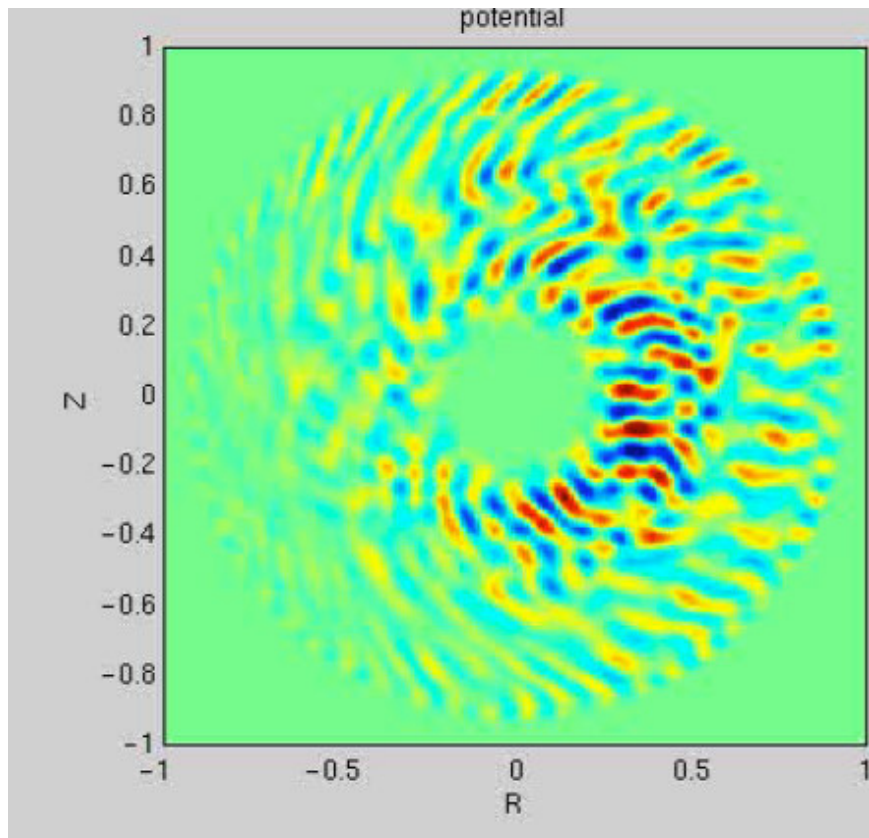
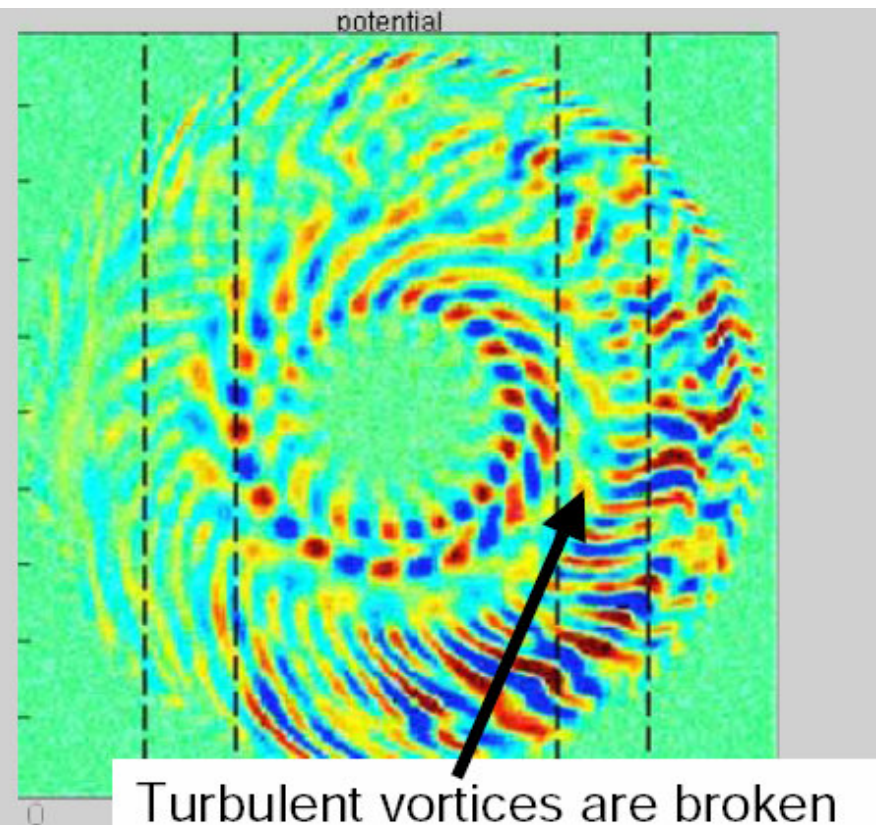


Figure 1. Line average density traces from an L-mode discharge (dubbed Type 'a' for historical reasons) and an early H-mode discharge (Type 'b'). The window of NBI heating is indicated. The configuration was SN^+ (ion-grad- B drift towards the X-point).

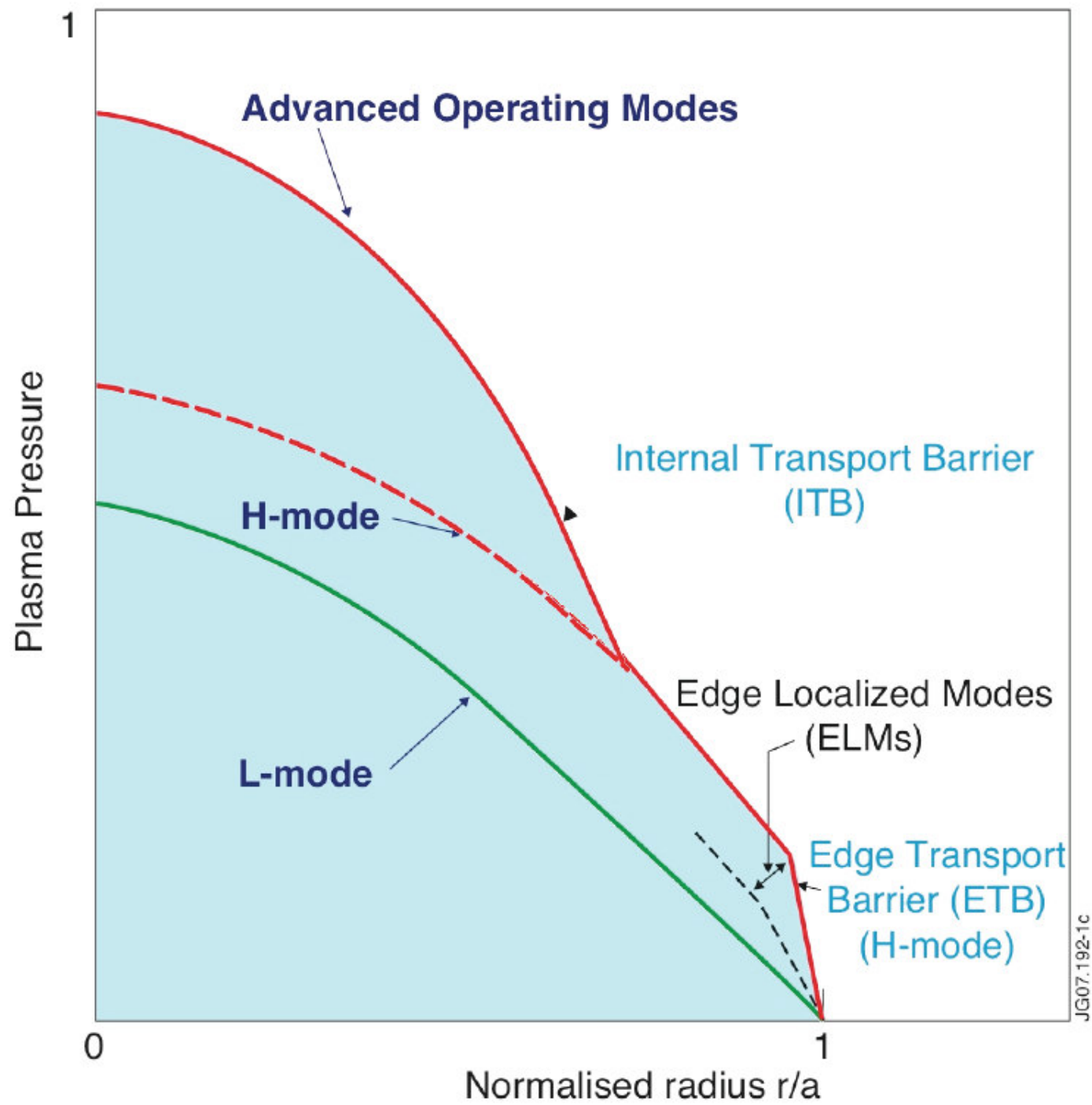
H-mode $\Rightarrow$ AOM mode



H-mode



Advanced Operating Mode



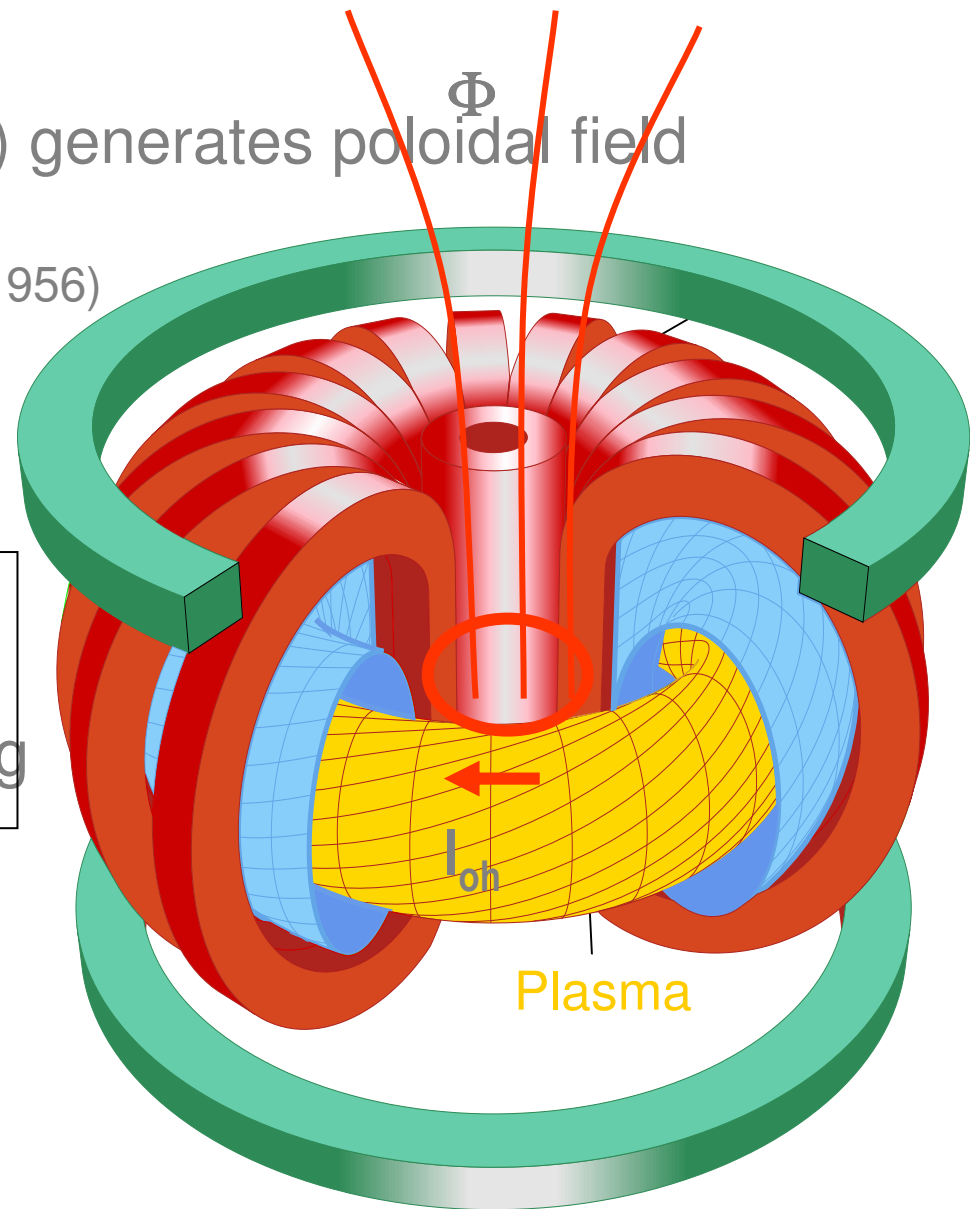
CONCEPT I, tokamak

61

large plasma current (MA) generates poloidal field component

Artsimovich, Sacharov (Moscow, 1956)

plasma current
inductively driven:
plasma transformer winding

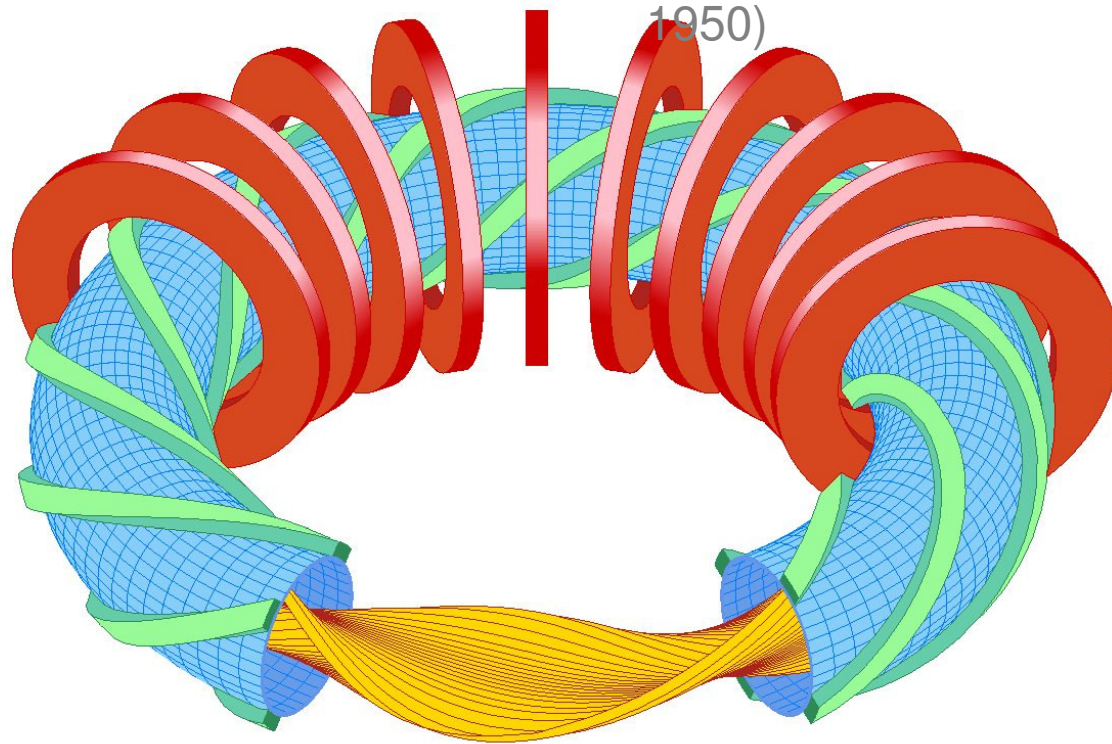


CONCEPT II, stellarator

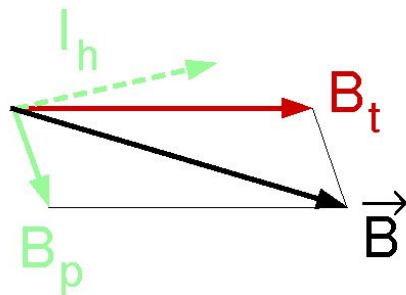
helical external coils create poloidal field

L. Spitzer jr (Princeton,

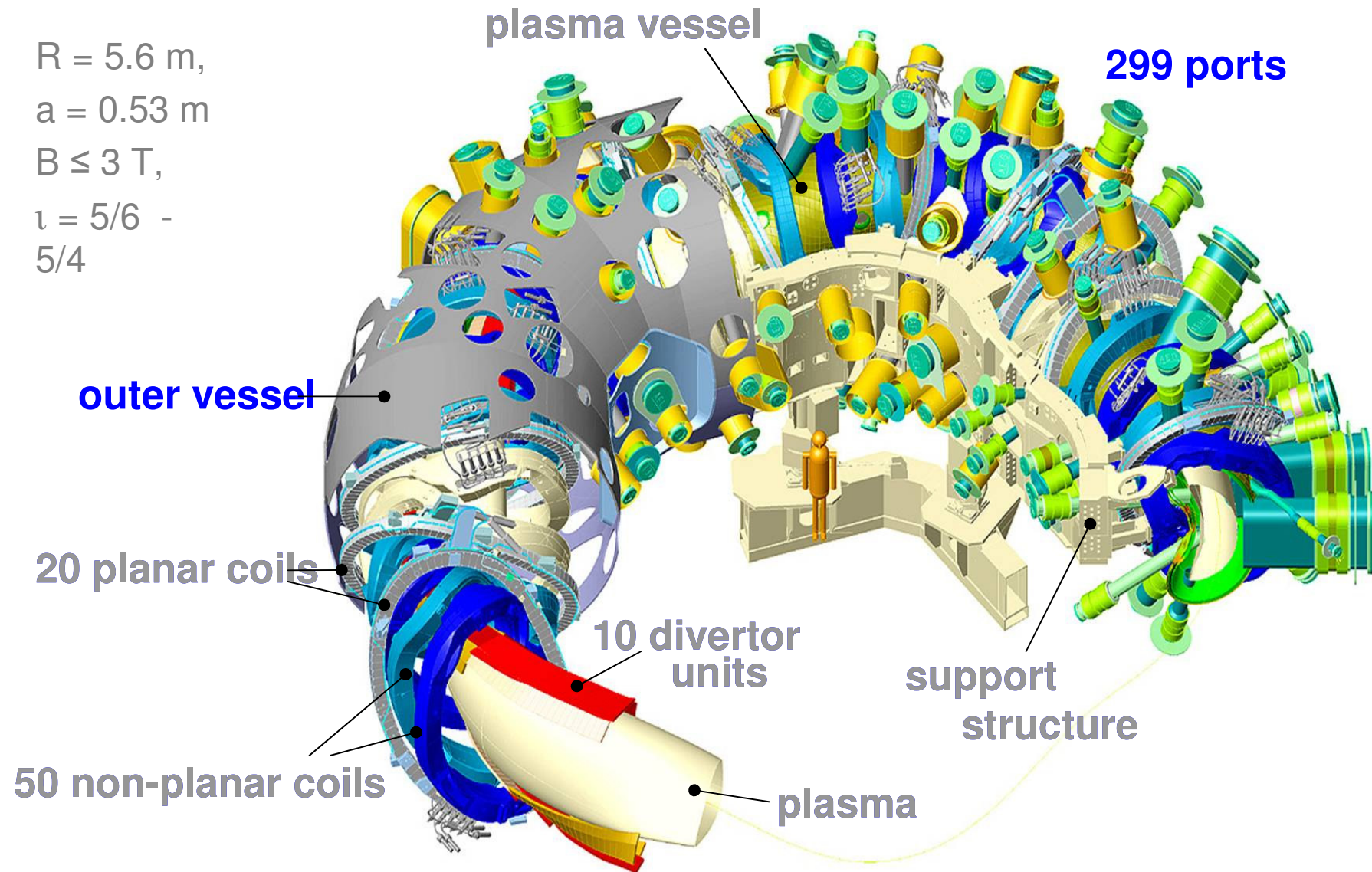
1950)



only external currents
no plasma current
intrinsically steady state



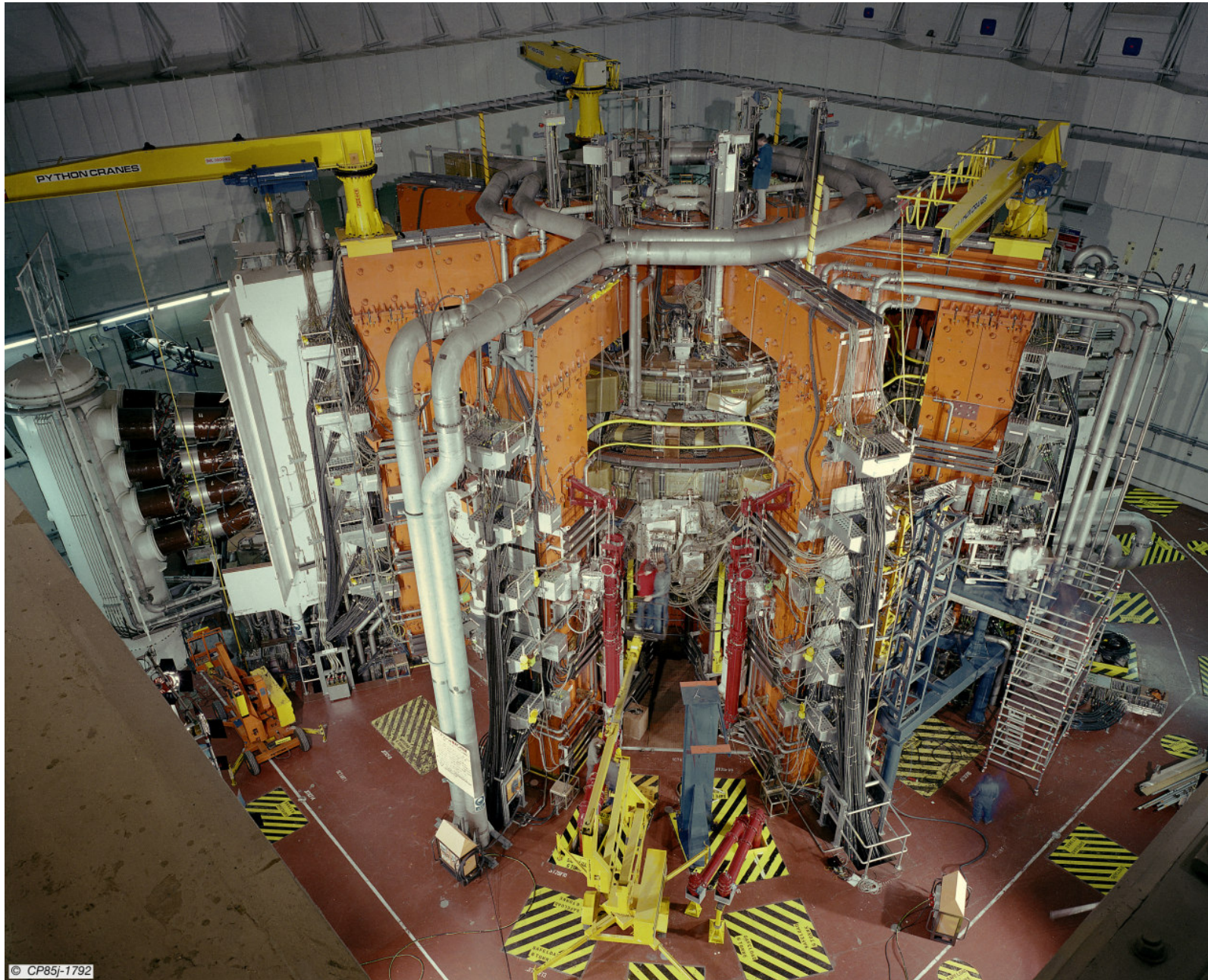
W7-X, components assembly



spectrometrer parameters

Line	B V	C VI	N VII	O VIII
Line wavelength (nm)	4.859	3.373	2.478	1.897
Wavelength range	4.70-5.20	3.10-3.50	2.25-2.65	1.85-2.00
Crystal	MLM	MLM	MLM	TIAP
2d (nm)	10.24	8.12	5.05	2.59
Incidence angle (deg)	28.33	24.54	29.36	47.09
Rowland circle radius (mm)	750	890	450	313
Cylindrical curvature	1 500	1 780	900	626
Arm length	712	739	441	459

Tokamak JET

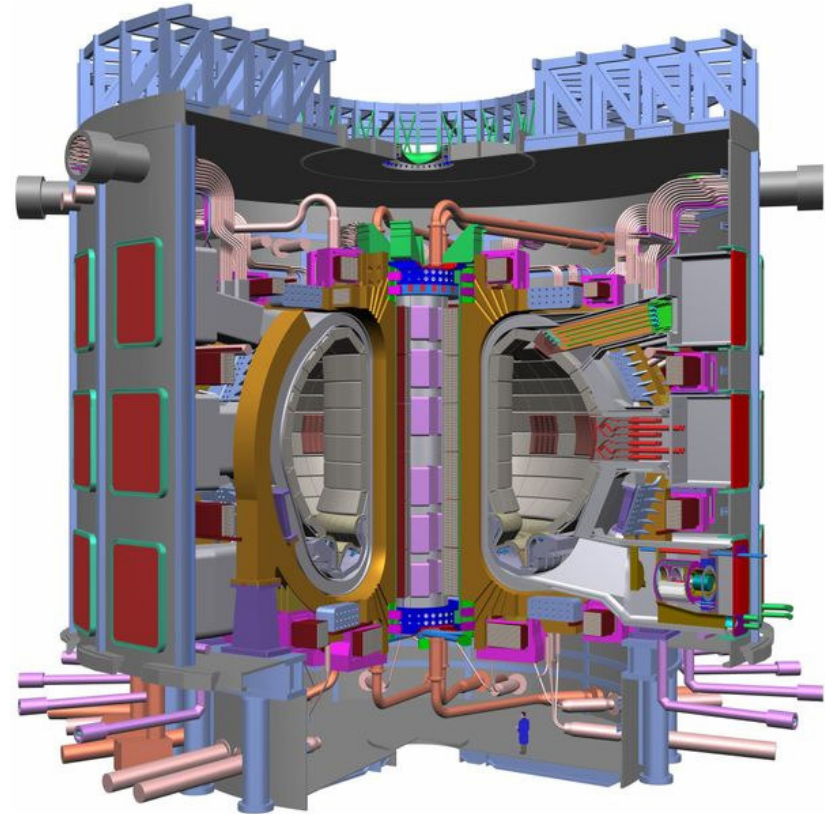


Czym jest/będzie ITER?

International
Thermonuclear
Experimental
Reactor

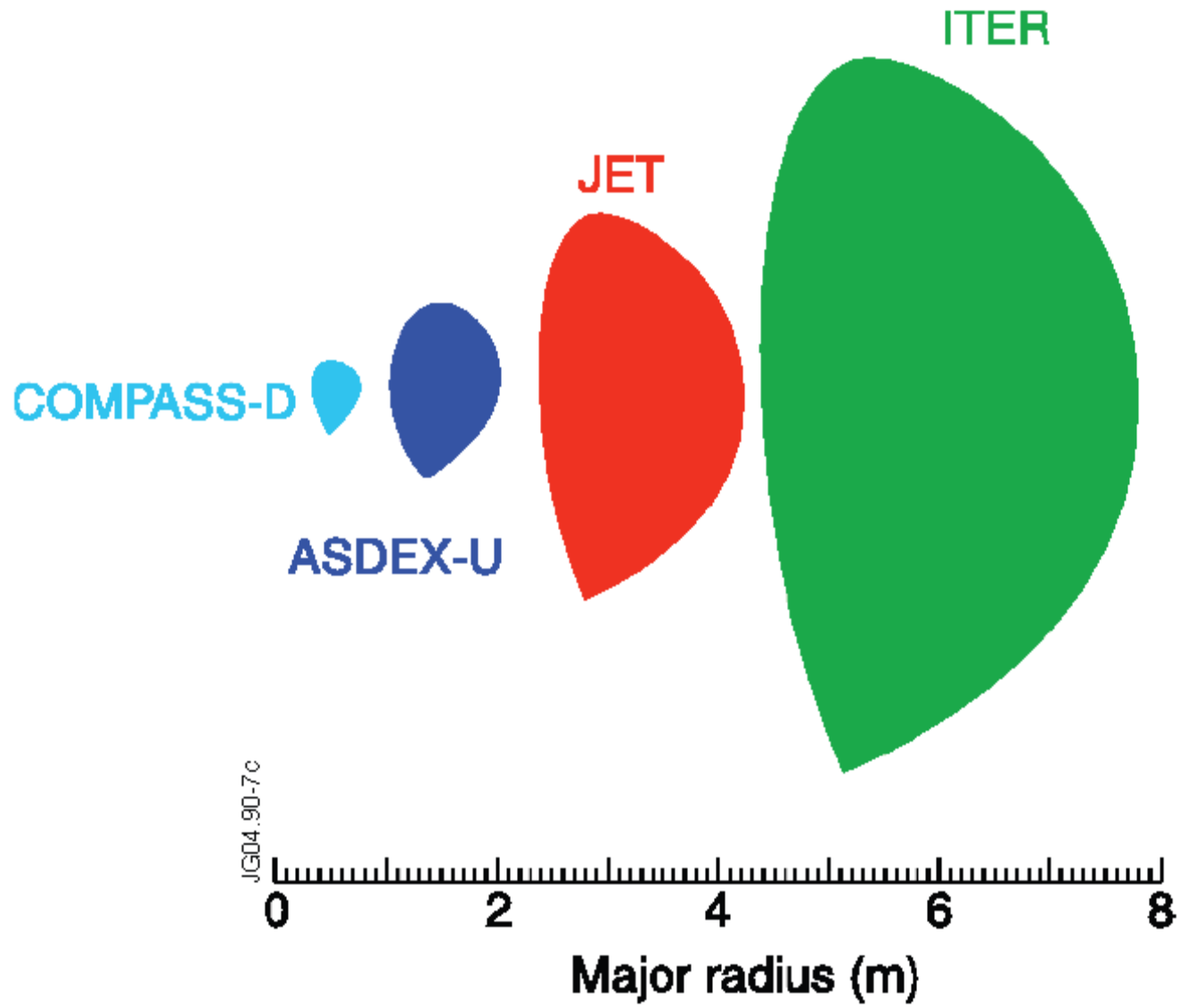
ITER

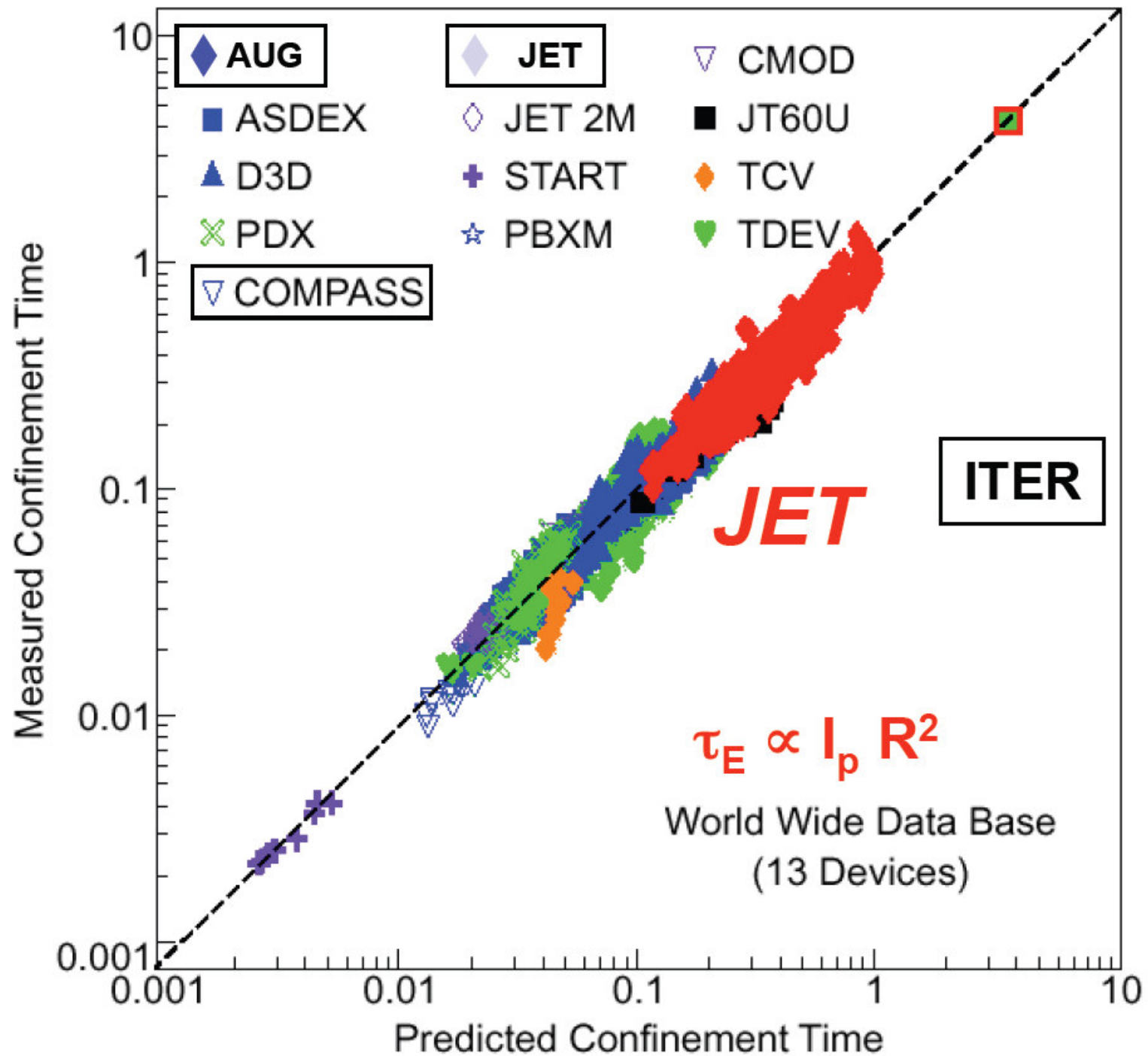
po łacinie oznacza „droga”



Celem eksperymentu ITER jest badanie plazmy w warunkach zbliżonych do tych jakie będą występować w reaktorze elektrowni termojądrowej.

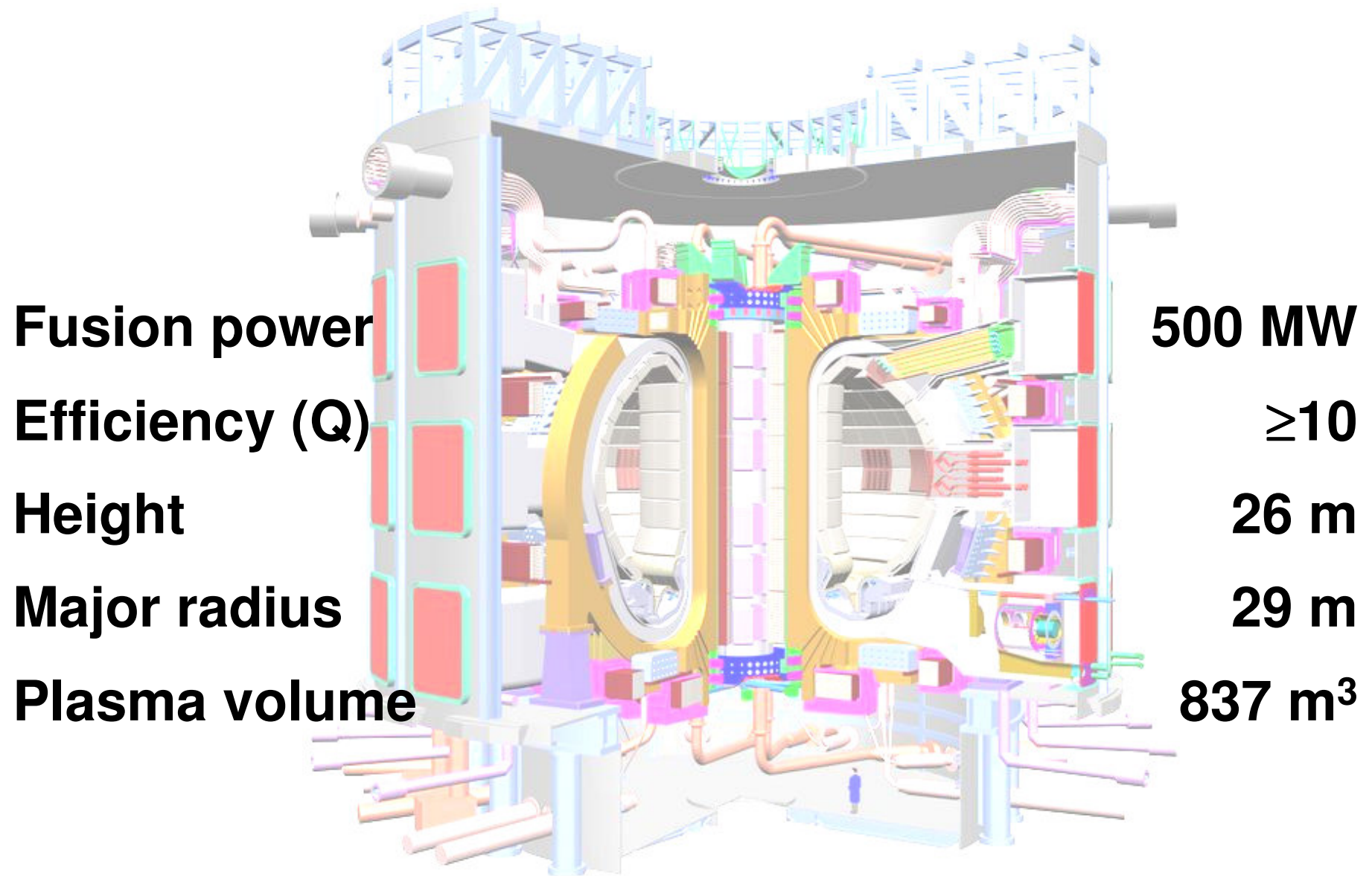






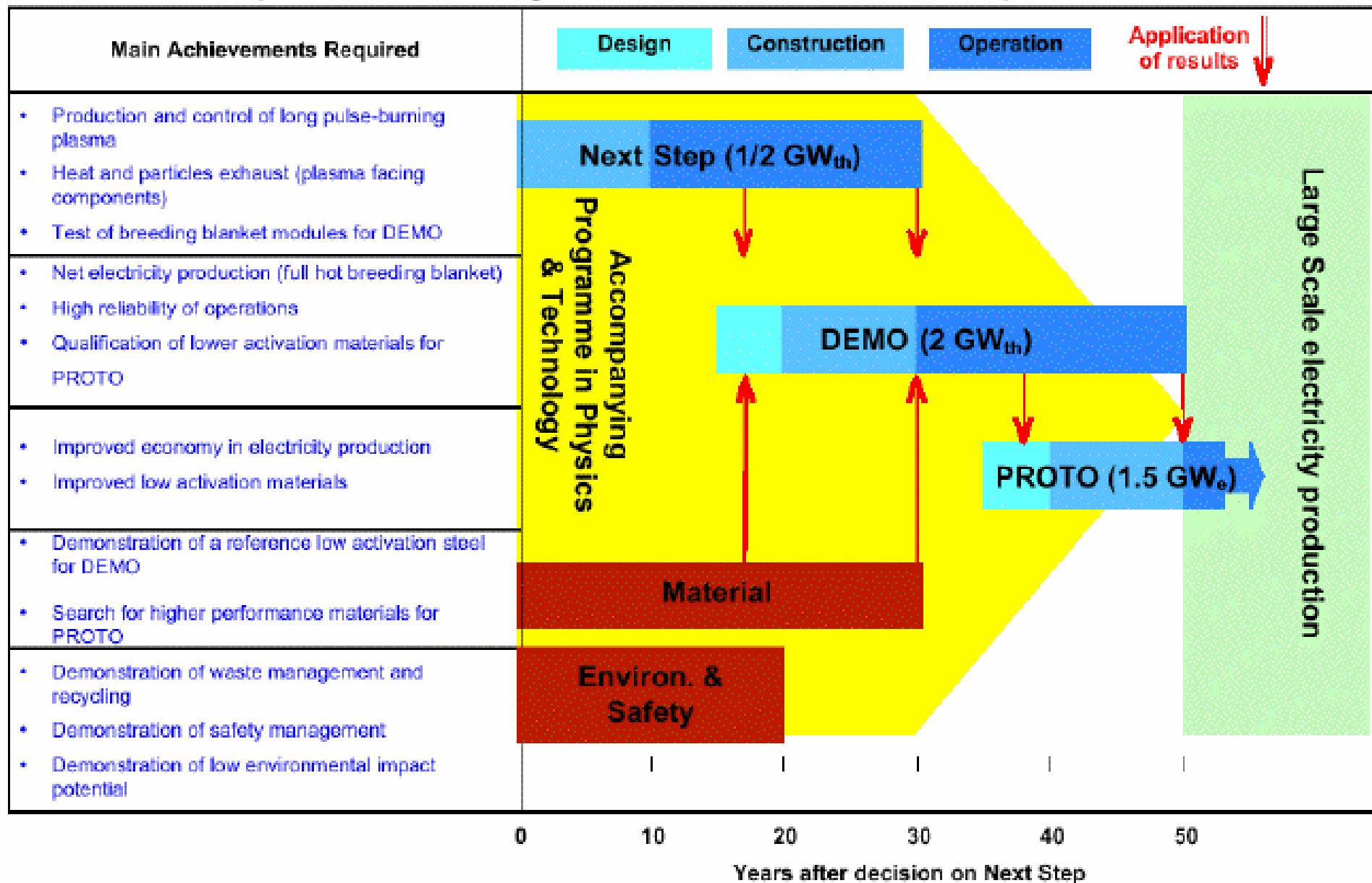
ITER

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Tentative Roadmap of Achievements starting from the decision to construct the Next Step

20.11.2001



Extracted from:

"Five Year Assessment Report related to the specific programme:
Nuclear energy covering the period 1995-1999" June 2000

Wendelstein 7-X

Under construction in
Greifswald, Germany

W7-X torus hall

ECRH



NBI, cryogenics

diagnostics, control room

assembly, ICRH



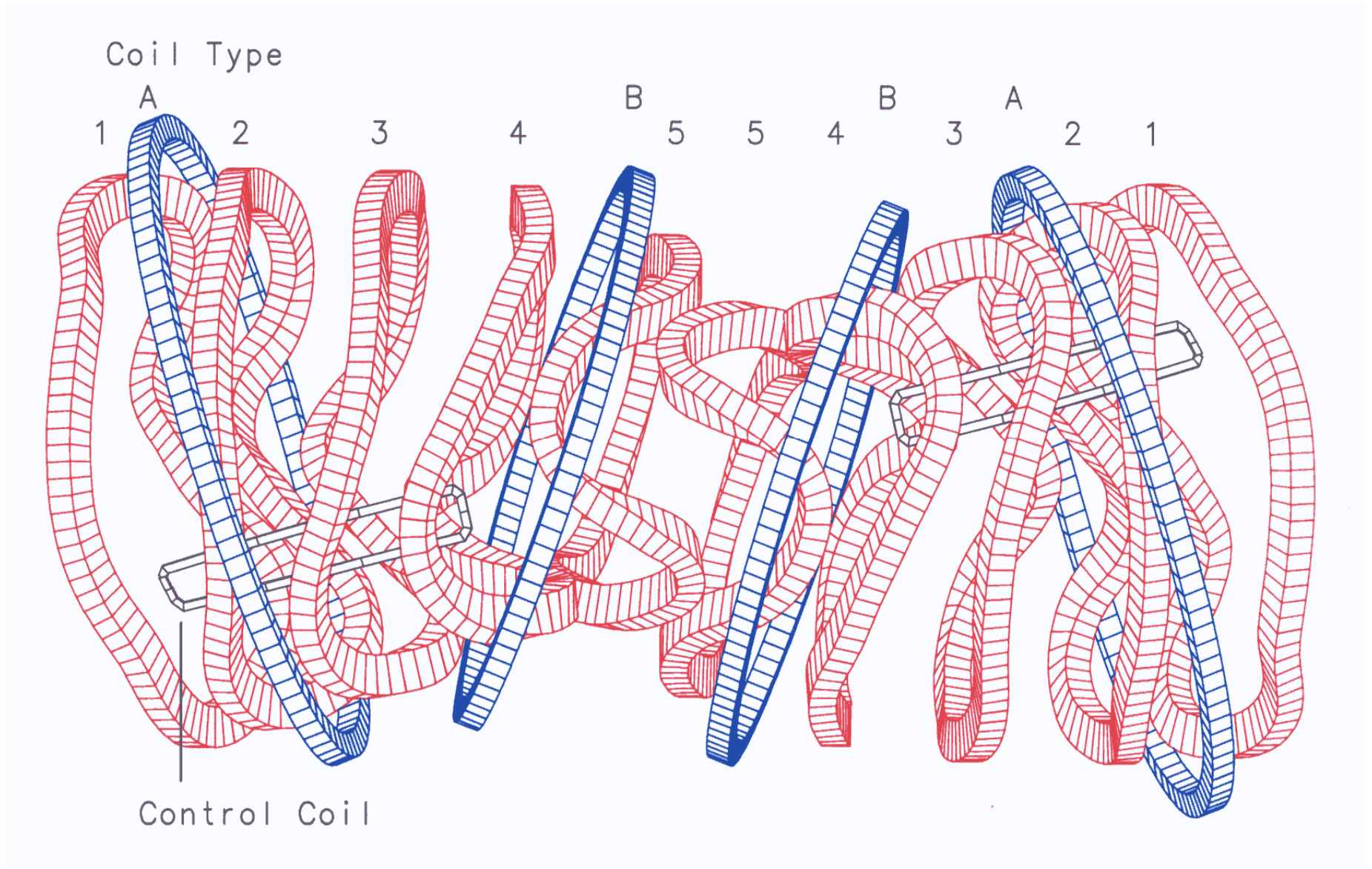
TEST ASSEMBLY, 3 coils mounted

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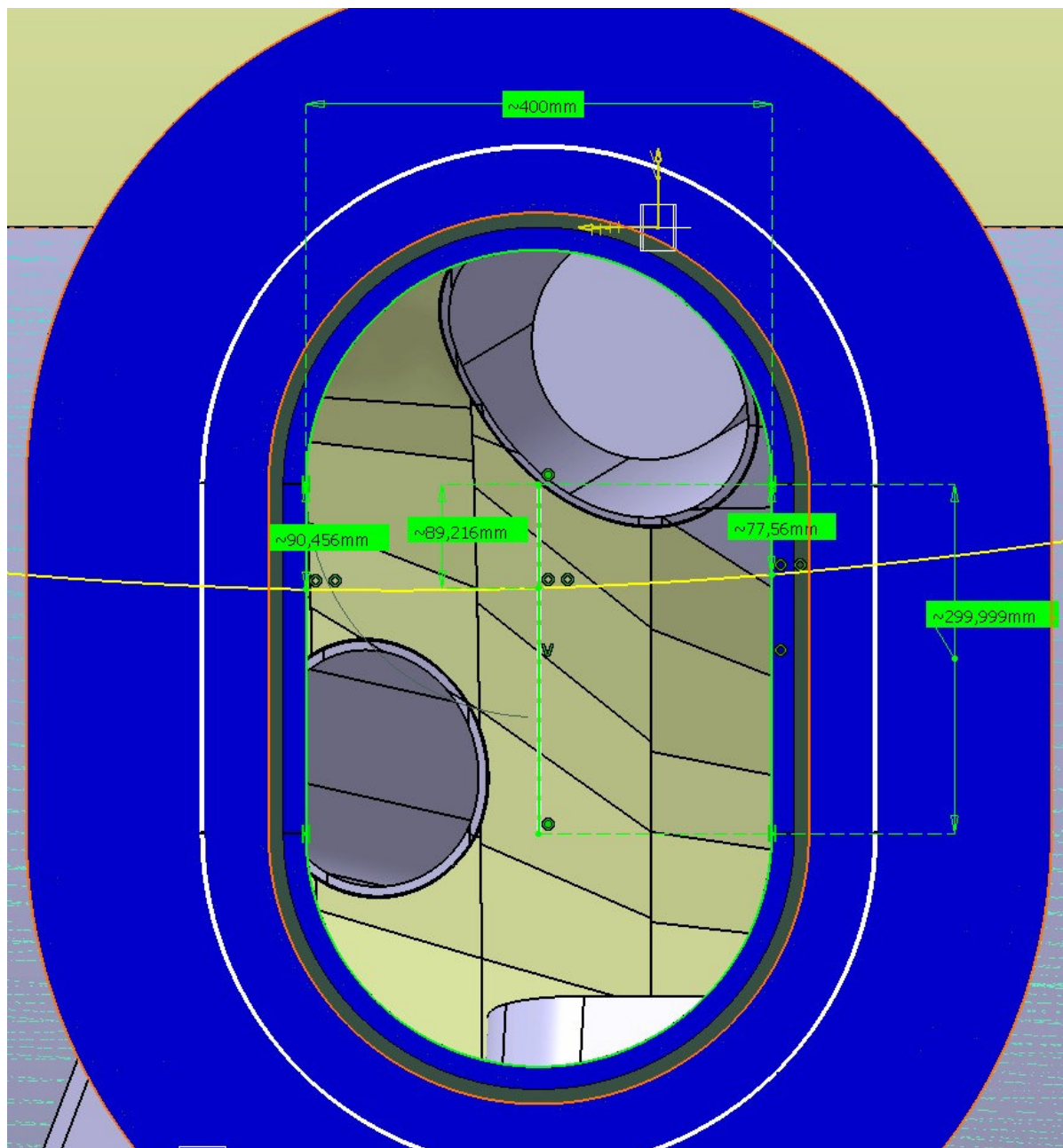


W7-X COILS, $2 \times (2 \text{ planar} + 5 \text{ non-planar})/\text{module}$

75

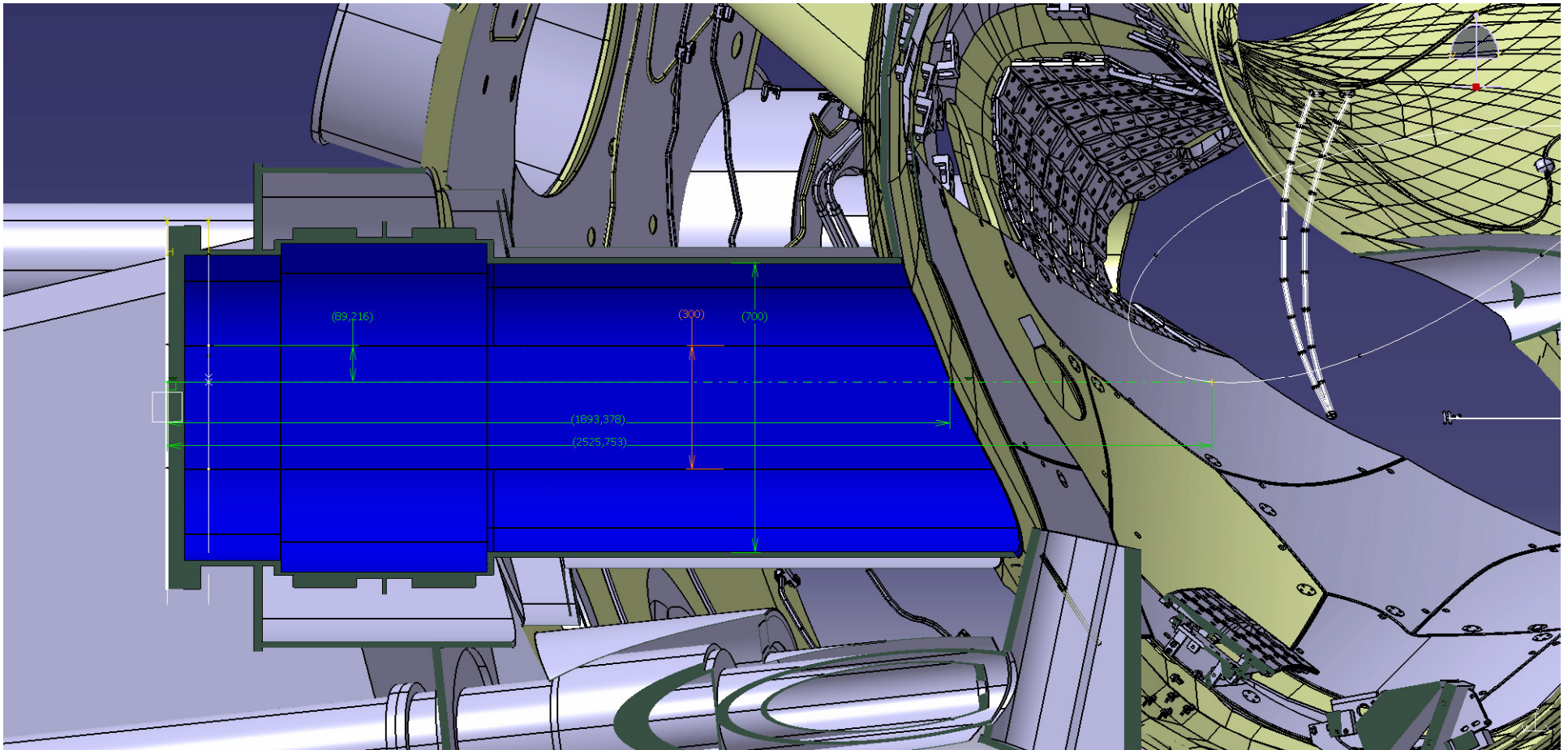


Port AEK30



Port AEK30

77



Port AEK30

78

